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MOD-5: A COMPUTER CODE FOR CALCULATIONS
OF NEUTRON TIME-ENERGY DISTRIBUTIONS
IN THE SLOWING DOWN REGION

by

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ABSTRACT:

This document provides users' information for a computer code, MOD-5, which calculates the time and energy dependent evolution of the neutron density in homogeneous media following initiation of a pulsed neutron source of arbitrary energy distribution. The code is based on a discrete stochastic model of the neutron slowing down process developed by the author. Copies of the code and associated computer software are available through the Argonne Code Center, Argonne National Laboratory, 9700 South Cass Avenue, Argonne, Illinois 60439.

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I. PROGRAM DESCRIPTION

Introduction

MOD-5 is a FORTRAN IV computer code utilizing a discrete stochastic model to obtain numerical solutions to the space independent neutron slowing down equation. The nucleus of the model is a "population vector" whose components are defined over a set of discrete energy states and specify the fraction of the total neutron population lying in a given state at a given time. The population vector may be advanced forward in discrete time steps by multiplication into a suitably designed one-step transition matrix. The method is rather similar to models devised by Perkel (1) and by Ohanian and Daitch (2) but has several unique features which greatly extend the capability of this approach.

Detailed discussions of the model may be found in two papers which have been included in this report (3,4) as Appendicies A and B, and the reader is urged to be thoroughly familiar with these before attempting to use MOD-5. Some useful information is also available in a doctoral dissertation based on this model (5), however many significant changes have been made since the dissertation was completed.

Specifics of the model will not be examined further here. This report is concerned only with providing technical information (a) about the structure and operation of the code, and (b) the mechanics of setting up typical problems for analysis by MOD-5.

Classes of Problems that May be Studied with MOD-5

MOD-5 evaluates numerical representation of time and energy dependent solutions of the slowing down equation following initiation of some sort of pulsed source. In its present configuration the following source conditions are available in MOD-5:

- (a) Delta function in energy, delta function in time.
- (b) Fission spectrum in energy, delta function in time.
- (c) Arbitrary energy spectrum, delta function in time.
- (d) Delta function in energy, uniform distribution over some small time interval.

A variety of functions and parameters are calculated by MOD-5. Among the more important are:

- (a) Spectra (lethargy dependent neutron density) and energy moments at selected times following initiation of the source,
- (b) Time dependent neutron density and slowing down density at selected energies and time moments of these densities,
- (c) Time dependent distributions of times to capture, leakage, and first fission, and moments of these distributions,
- (d) Steady state central core neutron flux and leakage flux in detail and group averaged form, and
- (e) Miscellaneous parameters such as k_{eff} , capture probability, etc.

Organization of the Program

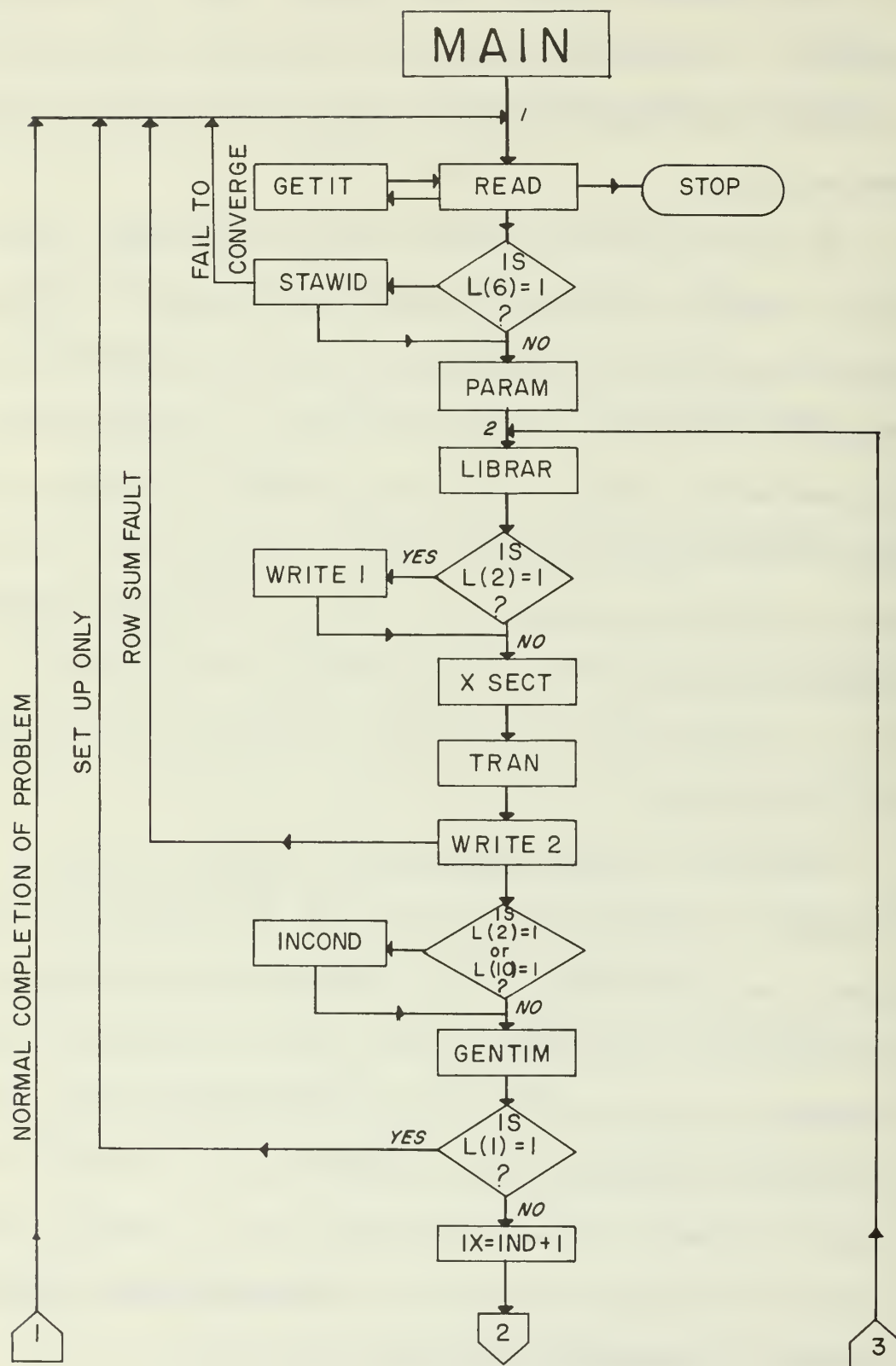
MOD-5 consists of a control program MAIN and 20 subroutines of

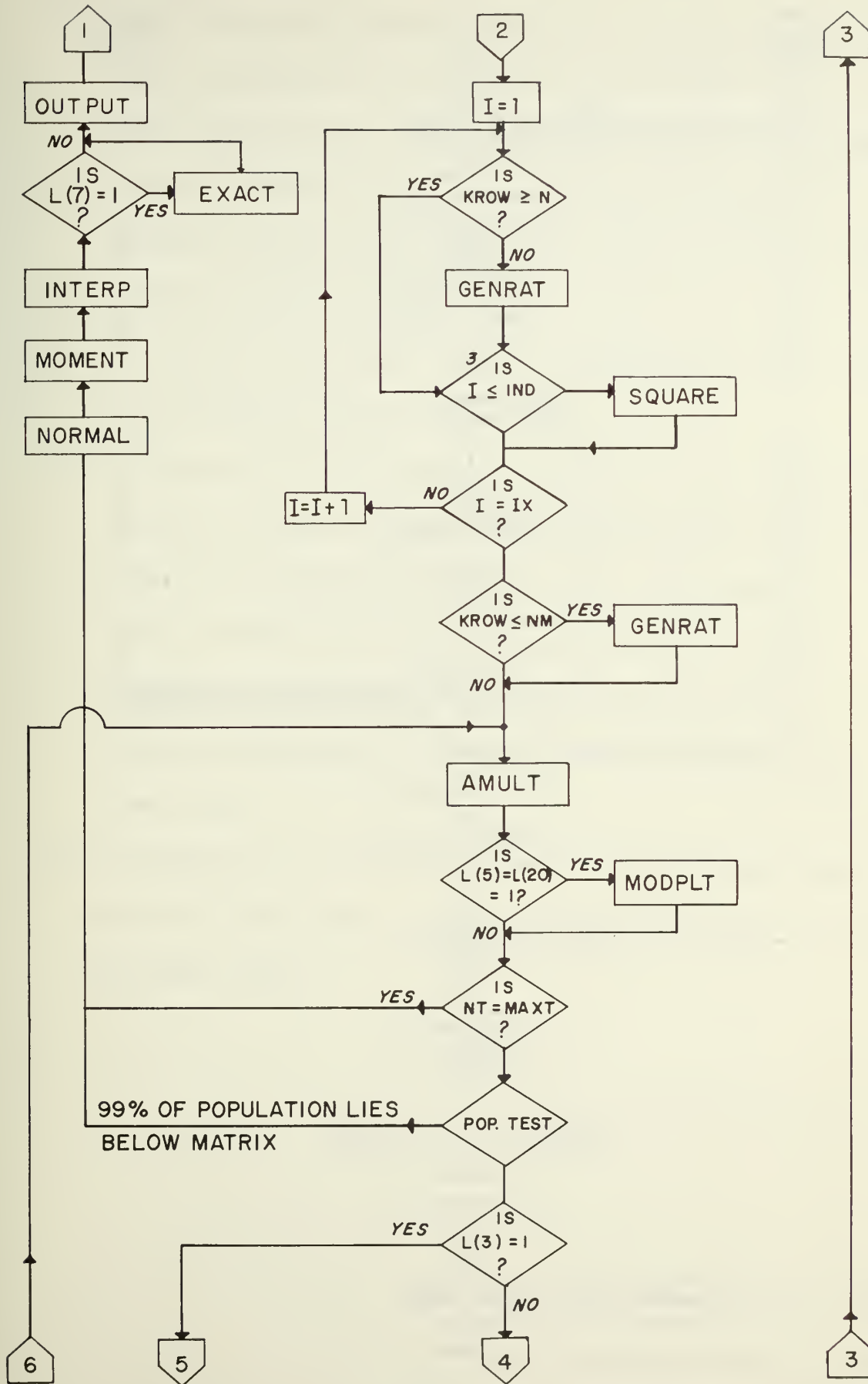
which all but one (GETIT) are called from MAIN. The flow of control is shown schematically in Figure 1 and a complete listing may be found in Appendix F. A description of the function of each subroutine may be found in Appendix E with elaborations in the comment cards included in the program listing.

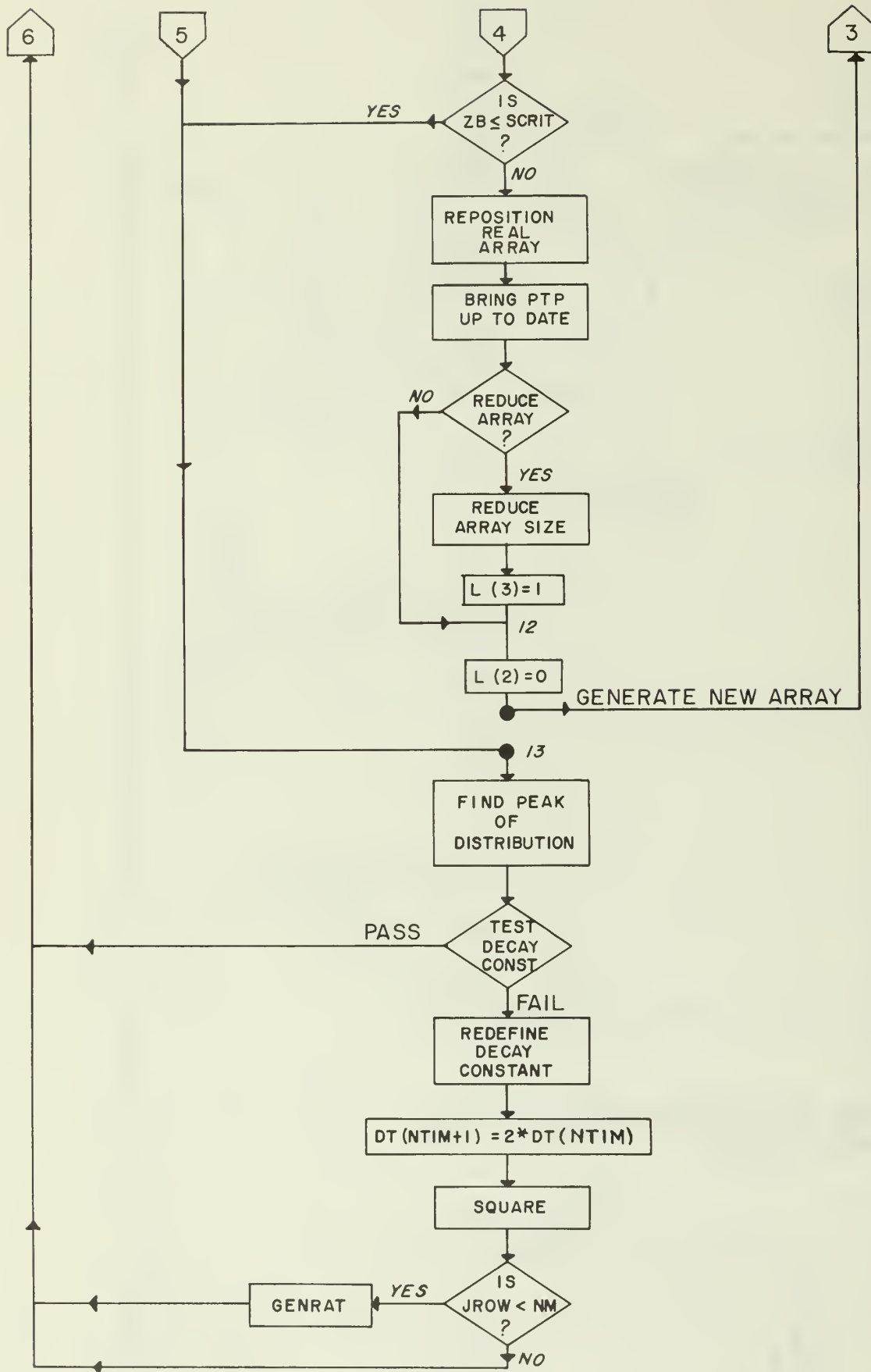
All input data enters via READ (on cards) or GETIT (disk or tape files of cross sections). Subroutines STAWID through INCOND in the calling sequence are concerned with processing cross sections and other data do define state parameters, transfer coefficients $p_{i,j}$, and initialization of the state vector. GENTIM selects appropriate transition matrix generating times and operating time steps (δt and Δt respectively in Eqs. 44-46 of App. A). SQUARE and GENRAT work together in a DO loop to generate a stepping matrix defined for Δt . AMULT multiplies the population vector into the stepping matrix to produce a new population vector defined for a time one step later.

After AMULT is called MAIN performs a series of operations to determine the progress of the neutron density, and selects one of several paths in the control sequence. If the desired number of operating iterations (multiplications of the population vector into the transition matrix) has been completed, or if at least 99% of the population lies below the energy range of interest or in capture, fission or leakage states, the evolution of the population vector is terminated and control goes to a series of routines that process the data for output listing (NORMAL-OUTPUT). Otherwise the matrix is evaluated for one of three possibilities:

FIGURE 1.







- (a) If the real operating matrix does not lie at the bottom of the virtual operating matrix (See App. B and C for definitions of these terms) the real matrix is evaluated to see whether it should be moved downward in energy. This shift is performed if a fraction SCRIT of the total population has been slowed down below the lower energy boundary of the real matrix. In this event the new position and size of the matrix is determined and then control is transferred to statement 2 in MAIN.
- (b) Next MAIN tests to see whether the operating time step should be doubled to compensate for decreasing collision rates as the population moves downward in energy. If it is to be doubled the matrix is squared by SQUARE. At this point new rows can be added to an incomplete matrix by GENRAT if the probability for a collision in Δt for each row ($v \Sigma_i \Delta t$) exceeds CRIT (DEFAULT value 0.001)
- (c) If the matrix is not shifted downward in energy, control is finally returned to AMULT for another operating iteration on the population vector.

II. USERS INFORMATION

Restrictions on Complexity of the Problem

Almost all of the variables in MOD-5 have been included in a COMMON block. The list below reflects the restrictions imposed by the COMMON block that appears in the program listing in Appendix F. The program has been adapted to handle up to 701 virtual states and 201 real states in a problem with a single isotope simply by replacing the existing COMMON blocks with an appropriately modified form.

Max. number of isotopes	5
Max. number of real energy states	71
Max. number of virtual energy states	201
Max. number of broad groups for input cross sections	26
Max. number of time steps	400
Max. number of time moments	21
Max. number of groups from which inelastic scattering can take place	10

Running Time

On the IBM 360 the execution time in seconds is given approximately by the formula

$$t = 0.004 N N_v + 10$$

where N is the number of real states and N_v is the number of virtual states

Basic Data Deck Format

The input data to MOD-5 requires a minimum of 6 data cards.

CARDS 1 - 3 Provide for a title block. FORMAT 18A4

CARD 4 A listing of initial values of the control variable L (Appendix D). Note that column 4 ($L(2)$) must always

have a "1" punch and column 24 (L(12)) must always have a "0" punch. FORMAT 30I2

CARD 5 A listing of numbers locating the desired isotopes in the external cross section file. The isotopes may be listed in any order at input but will be rearranged by subroutine READ in order of increasing atomic mass (exception: hydrogen is always placed last in the list).
FORMAT 5I5

(at this point in the program cross sections are retrieved from the disk or tape file)

CARDS 6 - n NAMELIST NUMBRS. A list of the variables that can be read in under NUMBRS can be found in the listing of subroutine READ. DEFAULT values of these variables may be found in the Glossary (Appendix C) or in READ. Note that the position of values assigned to such variables as RHO, SCALE, (i.e. all variables for which one dimension specifies the isotope) must be determined by the order of the isotope numbers on CARD 5.

Cross Sections

MOD-5 has been provided with a library of the Russian 26 group cross section set (6). Cross sections, inelastic scattering transfer matrices, and 300°K resonance self shielding coefficients are included for the 40 isotopes and 3 fission fragment sets listed below. The cross sections presumably will be stored by the user on disk or tape and are retrieved by subroutine GET IT. The numbers below are used on CARD 5 to specify the desired cross section sets. Auxiliary program SLOAD (Appendix G) should be used to load the external files in the correct format.

1. hydrogen	15. potassium	29. lead
2. deuterium	16. calcium	30. bismuth
3. lithium-6	17. titanium	31. thorium-232
4. lithium-7	18. vanadium	32. U-233
5. beryllium	19. chromium	33. U-234
6. boron-10	20. iron	34. U-235
7. boron-11	21. nickel	35. U-236
8. carbon	22. copper	36. U-238
9. nitrogen	23. zirconium	37. Pu-239
10. oxygen	24. niobium	38. Pu-240
11. sodium	25. molybdenum	39. Pu-241
12. magnisium	26. tantalum	40. Pu-242
13. aluminum	27. tungsten	41. U-233 Fiss.Frags.
14. silicon	28. rhenium	42. U-235 Fiss.Frags.
		43. Pu-239 Fiss.Frags.

Sample Problems

Portions of the output listings for the following examples may be found in Appendix H.

```

EXAMPLE NUMBER 1
EVALUATION OF ASYMPTOTIC SOLUTION TO SLOWING DOWN EQUATION.
(INFINITE GRAPHIC MODERATOR. DELTA SOURCE IN TIME AND ENERGY.
0 1 1 1 0 1 1 0 0 0 1 0 0 0 1
8
ENUMBERS N=51, NVIR=51, NM0=8, NMCM=1,2,3,4,5,-1,-2,-3, PHO=1.64,
(50)=1.00, SIGTE=26*4.7, SIGF3=26*0., SIGNR=26*0., SIGCR=26*0.,
SIGTE=26*4.7, NMUR=26*0.05556, F=144*0., FSS=3120*1., &END

```

Comments:

CARD 4	L(3)=1	because N=NVIR (CARD 6)
	L(4)=1	because it is desired to study the time dependence of the neutron density at E(NM)
	L(6)=1	The state widths will be chosen to provide an optimal description of elastic scattering. See description of STAWID in App. F.
	L(7)=1	To compare calculated and theoretical time moments.
	L(8)=L(9)=0	Because L(11)=1
	L(11)=1	Delta function source
	L(12)=0	Required input value, this variable is redefined internally

L(15)=1	A listing of the time dependent neutron density at EII(NM) and slowing down density at E(NM) will be printed in OUTPUT.
CARD 5 NOLIST(1)=8, NOLIST(2)=0,etc.	Cross sections for carbon will be retrieved from the external file. No other isotopes are present.
CARD 6 N=51 NVIR=51 NMO=8 RHO=1.64 E(50)=1.00 SIGTB=26*4.7, etc	<p>51 real states 51 virtual states 8 time moments will be evaluated to orders listed in NMOM Nominal mass density of graphite (gm/cc) Lowest energy of interest is 1.00 eV.</p> <p>Cross sections from external files are replaced with an idealized set in which there is only elastic scattering and the cross section is independent of energy.</p>

PERTINENT DEFAULT VALUES ASSUMED

NI=1 NF=0 NSCAT=1 MAXT=400 CRIT=.001	<p>one isotope no fissionable isotopes Stawid is called (because L(6)=1) and will optimize for scattering into one lower energy state. If necessary, the population vector can be evaluated for 400 time steps Normally user will not need to override DEFAULT values of CRIT OPCRIT, SQCRIT, SCRIT and CONV. The chosen values are based on operating experience.</p>
--	--

EXAMPLE NUMBER 2

STUDY OF SLOWING DOWN OF NEUTRONS FROM A 0.5 MICROSECOND, 2.46 MeV PULSED SOURCE (D,C). DEMONSTRATES TRAVELING ARRAY.

1 0 0 0 1 1 0 1 0 0 0 0 0 1 0 0 1

NUMBPS NVIR=109, NSCAT=2, OPCRIT=0.5, SQCRIT=0.25, TIMWID=0.5E-06, RHO=1.64, E(1)=2.46E-06, 8E-06

Comments:

CARD 4 L(3)=0

because $N < NVIR$, the real transition matrix will follow the pulse as it moves downward in energy.

L(4)=0	In this problem the evolution of the neutron distribution from the source is of special interest.
L(9)=1	The source simulates a $d(d,t)n$ pulsed source with a delta-function distribution in energy and a uniform 0.5 microsecond distribution in time (specified on CARD 6)
L(11)=0	Complementary to L(9)=1
L(18)=1	In addition to printing interpolated time distributions the program will print DENS and SDENS at every operating time step.
CARD 5	Same as Example 1
CARD 6 NSCAT=2	The distribution of neutrons following an elastic collision spans two lower energy states (optimized in STAWID)
NVIR=109	With this many states the energy range 2.46 MeV to 1.0 eV is spanned.
OPCRIT=0.50	
SQCRIT=0.25	Increasing the values of these parameters increases the operating time step widths proportionately. In this problem MAXT=400 is not adequate to allow a run to completion unless OPCRIT and SQCRIT are increased.
E(1)=2.46E 06	Since L(4) = 0, E(1) must be specified or a DEFAULT value of 10.5 MeV will be assigned.

The DEFAULT option for cross sections is to load from the external file.

```

EXAMPLE NUMBER 3
STUDY OF SLOWING DOWN SPECTRA IN REPRESENTATIVE FAST MULTIPLY-
ING ASSEMBLY. FISSION SOURCE - DELTA FUNCTION IN TIME.
0 1 0 0 1 0 0 0 0 1 0 0 0 0 0 1
 34 36 20 12
&NUMPRS NVIR=121,NI=4,NF=2,PHI=2.622,3.016,.965,.848,BUCKLE=0.0072,
BU(2)=119*.05,SCALE=52*.1,26*.5,&END

```

This problem analyzes various time dependent processes in a fast multiplying system. Steady state properties are derived by integrations over time

CARD 4 L(4)=0

L(6)=0

L(8)=L(9)=
L(11)=0

L(12)=0

CARD 5

CARDS 6-7 NI=4
NF=2
RHO(1)=2.622,
3.016,...

BUCKLE=0.0072
DU(2)=119*.05

E(1) must be specified, in this case a DEFAULT value of 10.5 MeV will be used since no other value is specified in NUMBRS (10.5 MeV is the upper energy limit of the 26 group cross section set).

STAWID is not called. Should L(6)=1 STAWID would be called and the state structure would be optimized for the isotope of smallest mass (generally also of the greatest moderating power)

By DEFAULT a fission spectrum source, delta function in time, will be defined.

This will be redefined to L(12)=1 in subroutine XSECT because there are inelastic scattering isotopes present. The first isotope on the input data set is number 34 in the external file (U-235), the second is number 36 (U-238), etc. READ will rearrange these in ascending numerical order.

Four isotopes in the system
Two of the isotopes are fissionable

The density of U-235 is 2.622 gm/cc, of U-238 is 3.016 gm/cc, etc. These numbers appear in an order consistent with the list on CARD 5 and will also be rearranged by READ.

Geometric buckling

Because STAWID is not called, the lethargy widths (or boundary values) must be specified for all states.

IMPORTANT: For the user's convenience the lethargy at input is specified to log base 10 and then converted to base e internally. Thus the specification DU=0.05 provides 20 states per decade. Note that although the state widths are uniform in these examples they need not be in general. DU(1)=0 and DU(N)=1.E06 by DEFAULT always and need not be specified.

SCALE(1)=26*0.1,
26*0.1,26*0.5

The mean lethargy increment per elastic collision is less than the state widths for U-235, U-238 and iron. Scale factors of 0.1, 0.1, and 0.5, respectively, ensure that the energy distribution of scattered neutrons extends into the next lower energy state. For aluminum the DEFAULT scale factor of 1.0 is assumed.

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4. T. J. WILLIAMSON and R. W. ALBRECHT, Nucl. Sci. Engr., 42, 89-111 (1970).
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6. L. P. ABAGYAN, N. O. BAZAZYANTS, I. I. BONDARENKO and M. N. NIKOLAEV, Group Constants For Nuclear Reactor Calculations, Consultants Bureau, New York (1964).

Other Related References

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APPENDIX A

(pages 19 - 36 inclusive)

APPENDIX B

(pages 38 - 42 inclusive)

Technical Notes

Calculations of Neutron Time-Energy Distributions in Heavy Moderators

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The task of making detailed neutronics calculations for heavy moderating isotopes has always been a most difficult problem for a variety of reasons. Consider, for instance, the problem of calculating the time and energy (or lethargy) dependent neutron density $N(E, t)$ that develops in a homogeneous moderator during and following the introduction of a source, $S(E, t)$ in the MeV range. Monte Carlo techniques are very impractical because of the large number of collisions required per history. The usual multigroup stepping matrix techniques¹ are inapplicable for at least two reasons: (a) inordinately large matrices are required when collision energy losses are small and (b) scattering through the inelastic region may take but a microsecond while moderation to thermal energies may require times of the order of a millisecond, thus making it virtually impossible to choose a suitable time step upon which to define the matrix.

Analytical techniques^{2,3} have been reasonably successful in predicting the low energy or asymptotic shape of $N(E, t)$ but they are incapable of dealing with the complexity of inelastic scattering, the effects of which carry over to relatively low energies.

We have been able to overcome many of these difficulties using our recently developed discrete stochastic model.⁴ The model is similar in many respects to the usual time-dependent multigroup theory in which a multigroup neutron spectrum, represented here by a state vector $\bar{s}(t)$

$$\bar{s}(t) = [s_1(t), s_2(t), \dots] \quad (1)$$

is stepped forward in time by repeated multiplication into a stepping matrix $\bar{P}_{\Delta t}$

$$\bar{s}(t + \Delta t) = \bar{s}(t) \cdot \bar{P}_{\Delta t}, \quad (2)$$

where the elements $P_{i,j}(\Delta t)$ of $\bar{P}_{\Delta t}$ are probabilities for transition between groups i and j during the interval Δt , and $s_i(t)$ is the population of group i . The new model differs from multigroup theory in the method of calculating transition probabilities and in the use of a well-known result of the theory of Markov processes which allows the stepping interval to be conveniently increased

$$\bar{P}_{n\Delta t} = (\bar{P}_{\Delta t})^n \quad (3)$$

Equation (3) provides a mechanism for the generation of a stepping matrix with transition probabilities of consistent accuracy for any step and energy range.

A requirement for maximum accuracy in multigroup calculations of elastic scattering processes is that the groups be small enough to allow neutrons to scatter from the mean scattering energy in one group to the smallest energy of the next lower group. For moderating isotopes with mass $A \sim 200$ this requires at least 160 groups per decade in energy. Unfortunately, on most computers a stepping matrix of dimension 200×200 represents a practical upper limit (both in terms of computation time and storage requirements) while typical slowing down problems span four or more decades.

We have circumvented this restriction by allowing the stepping matrix to cover only that region of energy occupied by the neutron population at a given instant. It is a well-known result of collision theory⁵ that with purely elastic scattering the asymptotic energy distribution of moderating neutrons is nearly Gaussian with a dispersion (relative standard deviation) no larger than

$$D = \left[\frac{\langle E^2 \rangle - \langle E \rangle^2}{\langle E \rangle^2} \right]^{1/2} = \left(\frac{8}{3A_L} \right)^{1/2}, \quad (4)$$

where A_L is the atomic mass of the lightest isotope present. As the neutron population moves downward in energy following a short burst from the source, we allow the matrix to move with it using the previously mentioned technique to generate new rows of transition probabilities at the bottom of the matrix, and to increase time steps as appropriate (Fig. 1). In the calculation described below, a real matrix of dimension 200×200 was carried down the diagonal of a "virtual" (elements not stored in computer) matrix of dimension 700×700 .

To reduce calculation times we have used the fact that in the elastic scattering region, transition probabilities decrease geometrically along a row moving away from the diagonal. As a result, for almost any useful time step and any moderator with $A > 10$, the transition probability

¹A. K. GHATAK and H. C. HONECK, *Nucl. Sci. Eng.*, **21**, 227 (1965).

²R. E. MARSHAK, *Rev. Mod. Phys.*, **19**, 185 (1947).

³A. A. BERGMAN et al., *Proc. First Intern. Conf. Peaceful Uses At. Energy*, **4**, 135 (1955).

⁴T. J. WILLIAMSON and R. W. ALBRECHT, *Nucl. Sci. Eng.*, **37**, 41 (1969).

⁵K. H. BECKURTS and K. WIRTZ, *Neutron Physics*, p. 174, Springer-Verlag, New York (1964).

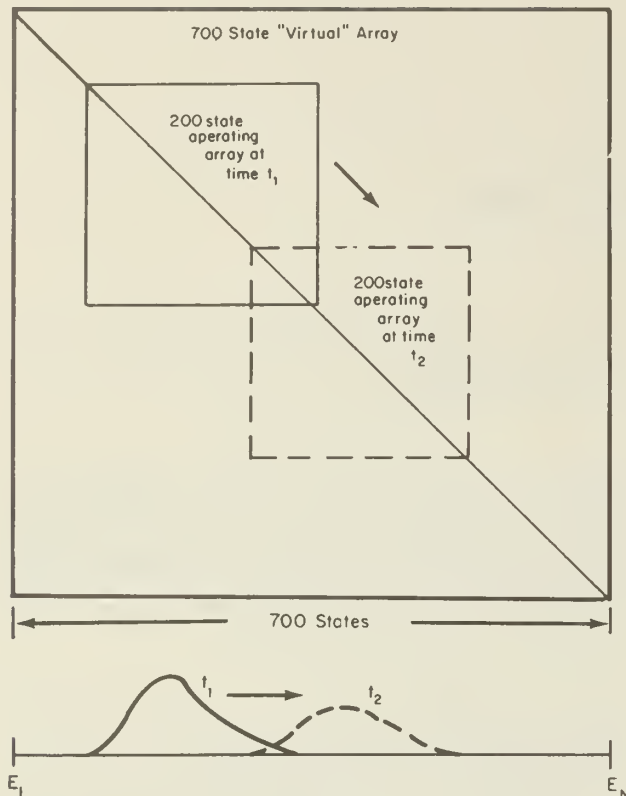


Fig. 1. Scheme for allowing transition matrix to follow neutron population.

$P_{i,j}(\Delta t)$ becomes vanishingly small (say $< 10^{-6}$) if $j > i + 10$. Thus, when calculations are being performed it is unnecessary to include transitions to a state j from any state i that is more than 10 states above j .

Application of our model does require two important compromises. The lesser of the two is the assumption of a normal mode space dependence. This is somewhat justified by the long mean-free-paths characteristic of heavy moderators. The more important compromise is made necessary because of the cooperative effect of inelastic scattering and time distributed sources in dispersing the neutron distribution. Inelastic scattering alone will cause neutrons from a 14 MeV source to be spread out to below 100 keV within a few nanoseconds. Once they are below the inelastic threshold the neutrons lose energy far more slowly and the narrow asymptotic shape is developed after a few collisions. However, because of the initial dispersal of neutrons over the broad inelastic region it is impossible to provide a matrix large enough to treat elastic scattering properly. Instead a few relatively broad groups must be used to span the energy region in which inelastic scattering reactions are the dominant moderating process. Within this region the elastic scattering kernel is adjusted by a scaling process that has been described elsewhere.⁴ We have found that scaling tends to artificially disperse the energy distribution without significantly altering the mean energy. Since the inelastic kernel overwhelmingly dominates the shape of the distribution, scaling of elastic cross sections introduces only very minor errors.

As an example of the application of the model to heavy moderators we have chosen to study the lead slowing down spectrometer. Such devices have been in use for several

years, in Russia,³ in Germany,^{6,7} and elsewhere. Lead spectrometers usually have a simple configuration such as a large lead cube resting on a concrete pedestal and penetrated by a 14 MeV pulsed neutron source and a small sample channel. If the source pulses are of short duration the neutrons will eventually develop a relatively narrow distribution in energy and the mean energy of the distribution can be easily related to elapsed time. By observing the time dependence of the emission of capture gammas from the sample being studied one can deduce the energy dependence of cross sections. Popov and others have made a number of studies of (n,γ) cross sections by this means.⁸⁻¹¹ More recently, there has been active study of temperature effects on the energy dependence of (n,γ) cross sections of various fast reactor materials.⁷ Such studies are considered a necessary adjunct to the analysis of integral Doppler experiments and the theoretical analyses of uncertainties in nuclear data.

Two physical properties of a given spectrometer must be known before it is possible to extract cross sections from reaction rates and determine the accuracy of the estimates.

First it is necessary to know the mean energy of the neutrons as a function of the time elapsed from initiation of the neutron burst. It is easy to show that with a source that is a δ -function in time and velocity, and with a purely scattering medium of atomic mass A and scattering cross section Σ_s , the mean energy $\langle E(t) \rangle$ is given by^{3,7}

$$\langle E(t) \rangle = \frac{M_N A^2}{2 \Sigma_s^2} \frac{1}{\left(t + \frac{A}{v_0 \Sigma_s} \right)^2}, \quad (5)$$

where v_0 is the source neutron velocity and M_N is the neutron mass. Although the above result is based on a highly idealized model, our calculations have shown that it works very well over the entire energy range below the inelastic threshold. With realistic source burst widths of up to 4 μ sec the formula has been found to be satisfactory if t is replaced by elapsed time from mean emission time.

A far more difficult problem is posed by the desire to know something about the energy or lethargy distribution of the neutrons as time elapses. For most purposes it is sufficient to determine the relative standard deviation in energy (also commonly referred to as dispersion, or resolution) of $N(E,t)$ as a function of either the mean energy or the elapsed time. Clearly this parameter will determine the energy resolution of measured cross sections. Seufert and Stegemann⁷ have concluded that for their Doppler measurements it is necessary that

$$\frac{\Gamma_r}{\langle E \rangle} \ll \frac{\overline{D^2}}{\langle E \rangle} \ll D \ll 1, \quad (6)$$

where $\overline{D^2}$ is the mean level spacing, Γ_r is the gamma width of the resonances, and D is the dispersion of the neutrons [Eq. (4)].

⁶H. SEUFERT, "Untersuchung des Dopplereffektes in schnellen Neutronenspektren nach neuen experimentellen Methoden," Dissertation, Karlsruhe (1968).

⁷H. SEUFERT and D. STEGEMANN, Institute für Neutronenphysik und Reaktortechnik, KFK-631, Kernforschungszentrum, Karlsruhe (1967).

⁸A. I. ISAKOV, YU. P. POPOV, and F. L. SHAPIRO, *J. Exptl. Theor. Phys.*, (USSR), **38**, 989 (1960).

⁹N. T. KASHUKKEEV, YU. P. POPOV, and F. L. SHAPIRO, *J. Nucl. Energy*, **14**, 76 (1961).

¹⁰YU. P. POPOV and F. L. SHAPIRO, *Sov. Phys. JETP*, **15**, 683 (1962).

¹¹S. A. ROMANOV and F. L. SHAPIRO, *Sov. J. Nucl. Phys.*, **1**, 2, 159 (1965).

Our calculations have employed the 26-group ABN cross-section set¹² with an additional group added to cover the region from 10.5 to 14 MeV. The few resonances that occur in lead were ignored since they will not affect dispersion calculations significantly. The broad group cross sections were distributed over a fine group structure consisting of about 14 groups per decade above 300 keV and about 159 groups per decade below 300 keV. A geometric buckling of $B^2 = 0.000558$ corresponds to the physical dimensions of a large spectrometer (2.3 m on a side) but ignores such complications as the albedo of the concrete pedestal it sits on.

Output from the calculations consisted of (a) the neutron density in lethargy $N(u, t)$ and the relative standard deviation in energy and velocity at selected times following the initiation of the source, (b) time-dependent capture and leakage rates, and (c) steady-state flux.

The calculation was terminated when the neutrons reached 20 eV, at which point 75.7% had been lost to leakage, 12.2% had been captured in the lead and 12.1% were still slowing down.

Figure 2 displays the calculated lethargy-dependent neutron densities following source pulses of 0, 1, 2, and 4

μsec duration. Certain features of these figures are worth noting:

1. In Fig. 2a the structure and boundaries of the cross-section set are quite apparent. At 2 and 8 nsec the spectrum is almost completely determined by inelastic transfer from the source group.

2. In Figs. 2c and 2d a steady-state situation exists above 200 keV at the time the source is turned off.

3. A striking feature in all four cases is the rapidity with which a Gaussian shape is achieved by the population below 100 keV.

4. An estimate of the relative widths of the distributions at a given time is provided by the heights of the curves. Since the areas under the curves should be about the same at a given time (presuming the source is off) the width of the curve, or equivalently D , will vary inversely as the height. Thus at 50 μsec the distribution from a 4 μsec source is about 20% broader than that from a 1 μsec source.

A more complete description of the resolution of the pulses as a function of energy and elapsed time is given in Fig. 3. Here the abscissa is actually the ratio of D to its asymptotic value.

On the same figure we have plotted for comparison the

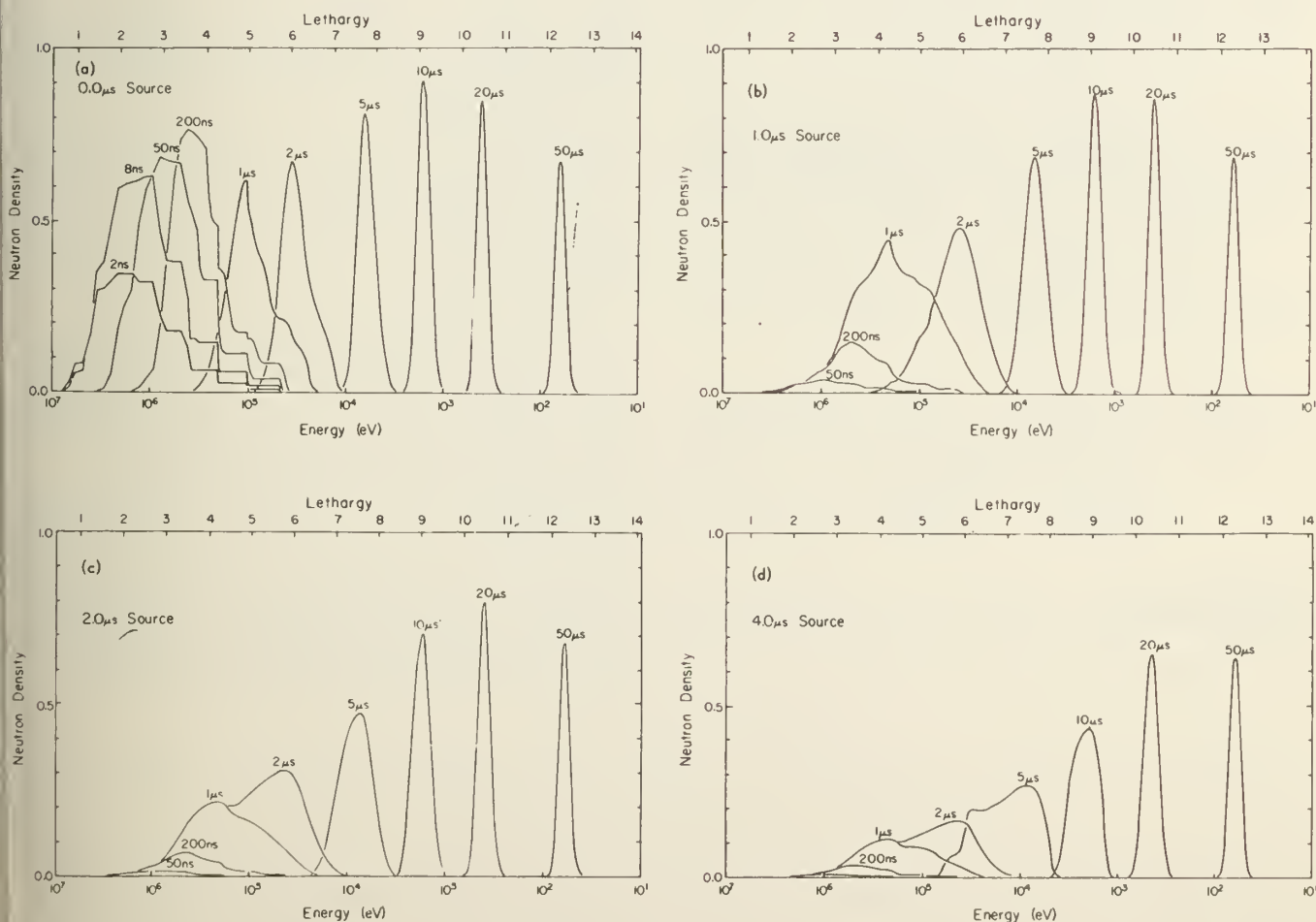


Fig. 2. The function $N(u, t)$ for source burst widths of 0, 1, 2, and 4 microseconds. Normalized to one source neutron at 14 MeV.

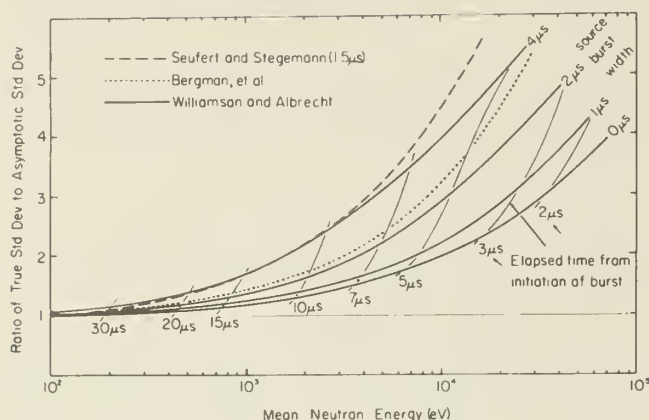


Fig. 3. Ratio of true to asymptotic value of relative standard deviation of $N(E,t)$ as a function of source burst width, mean energy, and time elapsed from initiation of source burst.

prediction given by Bergmann et al.³ (source time unspecified) and by Seufert for a 1.5 μsec source. Our results clearly provide a more optimistic estimate of the resolution of the spectrometer. Seufert's results are based on a model first proposed by Bergmann who suggested that the relative standard deviation in velocity ought to obey the following equation below 100 keV^a:

$$\left[\frac{\langle v^2 \rangle - \langle v \rangle^2}{\langle v \rangle^2} \right]^{1/2} = \left[\frac{2}{3A} \left(1 - \frac{\langle E \rangle}{E_0} \right) + D_0 \frac{\langle E \rangle}{E_0} \right]^{1/2}, \quad (7)$$

where $\langle E \rangle$ is the mean energy, $E_0 = 100$ keV and the constant D_0 was estimated to be ~ 0.3 . The theoretical basis for this expression is obscure. In fact, in light of recently developed expressions for the lethargy dependence of velocity moments we would expect a better formulation to

be a power series in $\left[\frac{\langle E \rangle}{E_0} \right]^{1/2}$.¹³ As a useful approximation however, the following expression provides results that compare well with calculations:

^aOur numerical calculations have shown that the following relation seems to hold true at all energies:

$$\frac{\frac{\langle v^2 \rangle - \langle v \rangle^2}{\langle v \rangle^2}}{\frac{2}{3A}} = \frac{\frac{\langle E^2 \rangle - \langle E \rangle^2}{\langle E \rangle^2}}{\frac{8}{3A}}.$$

The two quantities differ by no more than 5% above 100 keV and by less than 1% below 100 keV. So far, we have not obtained analytical verification of the result.

¹³T. J. WILLIAMSON, *Nucl. Sci. Eng.*, **39**, 273 (1970).

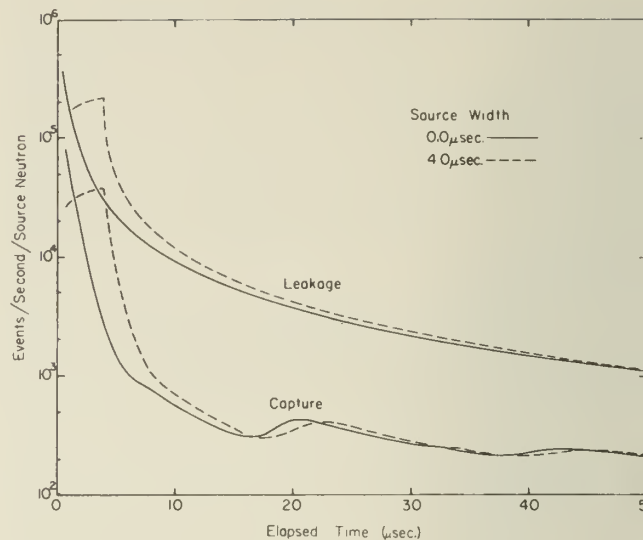


Fig. 4. Time dependence of leakage and capture for 0 and 4 μsec sources.

$$\left[\frac{\langle v^2 \rangle - \langle v \rangle^2}{\langle v \rangle^2} \right]^{1/2} = \left[\frac{2}{3A} + D_0 \left(\frac{\langle E \rangle}{E_0} \right)^{0.84} \right]^{1/2}, \quad (8)$$

where, again, $E_0 = 100$ keV, but D_0 is ~ 0.07 for source widths of less than 1 μsec. The much smaller value of D_0 is a consequence of the very rapid focusing of the distribution below the inelastic threshold, a fact that is certainly not intuitively obvious.

In practical applications of the spectrometer one of the overriding factors is the existence of background radiation mainly from capture gammas produced in the lead. In Fig. 4 the specific capture rate following 0 and 4 μsec bursts has been plotted. Consistent with reports of experimenters, a high background should persist for up to 5 μsec following the burst.

In closing, it should be noted that certain errors are inherent in the calculation and these are summarized below.

In a previous paper⁴ it was shown that generally multi-group calculated dispersions will be larger than the true values. With the optimized stochastic model D is about 5% too large at all energies; however, this error cancels out almost completely in the ratios plotted in Fig. 3.

The leakage rates in Fig. 4 are probably pessimistically large early in the history of the pulse. This is because neutrons are actually born near the center of the cube rather than in a fundamental mode. Clearly a neutron born at the center has a smaller probability for leakage during its lifetime than one born elsewhere. Because of the overestimate of leakage we would expect that the distribution in Fig. 2 should properly have slightly greater amplitude.

APPENDIX C

GLOSSARY OF FORTRAN VARIABLES USED IN MOD-5

Notes:

1. Numbers in parenthesis reference equations and Figures in the paper by Williamson and Albrecht (Appendix A).
2. If a variable does not appear in the common block the subroutine in which it does appear is given.
3. Jargon:
 - a) Group - A broad energy group as defined by cross section sets such as ABN
 - b) State - A fine "group" whose cross sections are typically drawn from the broad group set in which it lies.
 - c) Operating Iteration - The process of multiplying the state vector into the one step transition matrix which is thus called the "operating matrix (array)"
 - d) Generating Matrix - The matrix, typically defined for a very short time step, from which the operating matrix is derived by a sequence of squaring operations.
 - e) Real Matrix - The matrix of transition probabilities stored in core
 - f) Virtual Matrix - The actual matrix of transition probabilities this may be substantially larger than the Real Matrix but is not stored in core. For an explanation of the relationships between real and virtual matrices see the paper "Calculations of Neutron Time-Energy Distributions ..."
4. * Denotes variables which may be listed at input but for which default parameters have been defined in READ
5. ** Denotes variables which must be defined by the input data set and for which no meaningful default values are assigned.
6. *** Variables which must be updated if dimensions in the COMMON block are changed.

ACCUM	Total population outside real array
ALAM(k)	$N_{s,k}$ (Eq. A12)
ALP(k)	α'_k (Eq. 42)
ALPHA(k)	α_k
AMASS(k)**	A_k , atomic mass of isotope k
AMOM(i,j)	NMOM(i)-th time moment of distribution
AMU(i,m)*	Mean cosine of scattering angle for isotope m in state i (DEFAULT 0.0)
AMUB(k,m)	Group average cosine, group k, isotope m
ANUNOR(i)	Average neutron yield of fissions in state i
ATOM(2m-1,2m)	Name of isotope m
BGF(k)	Steady state fundamental mode flux in group k
BGSF(k)	Steady state leakage flux in group k
BUCKLE*	Buckling, B^2 (DEFAULT 0.0)
CAPT(n)	Neutron capture rate at time n
CCENER	Average fundamental mode energy in steady state
CONVC*	Convergence criteria for iterative calculation of optimized state widths in STAWID (DEFAULT 0.00001)
CORMOM(i,j)	NMOM(i)-th time moment corrected by extrapolation
CRIT*	Row generating criteria (45) (DEFAULT 0.001)
CSENER	Average energy of leakage spectrum neutrons
DB2(i)	Leakage cross section, DB^2 , in state i
D32B(k)	Leakage cross section in group k

DDT	Generating time δt (46)
DE(i)	Energy width of state i, $E(i-1) - E(i)$
DEABN(k)	Energy width of group k
DENS(n)	Neutron density in state NVM at time n
DINT(n,j)	Interpolated time distribution at time n
DIST(n,j)	Raw time distribution at time n
DT(m)	Stepping time after m-1 squarings of the ones step matrix, $DT(m) = DT(1) \cdot 2^{m-1}$
DTI	Time step for interpolated distributions
DTIM(n)	Time step width at time n
DU(i)**	$U(i-1) - U(i)$, Lethargy width of state i (DEFAULT $DU(1)=0.0$, $DU(N)=1.0$ E 06)
DUB(k)	Lethargy width of group k
DV(i)	$V(i-1) - V(i)$, velocity width of state i
E(i)*	Lower energy limit of state i (DEFAULT $E(1)=10.5$ E 06)
EABN(k)	Lower energy limit of group k
EFFK	k_{eff} with extrapolation correction
EII(i)	$\langle 1/E \rangle_i^{-1}$ (A5, A7)
EMIN(m)	E^{min} for isotope m, redefined for each state
ERR(i,j)	Difference between exact and calculated values of NMOM(i)-th moment of density and slowing down density in state NVM
ESTK	k_{eff} estimated from fission distribution without truncation correction

EXM(i,j)	Exact value of NMOM(i)-th moment of density or slowing down density. Based on predictions of Eriksson (ref. 3) and Kosaly and Nemeth for asymptotic distributions in single isotope systems with constant cross sections.
FISS(n)	Fission rate at time n
FLCOR(i)	Correction factor for flux in state i
FLUX(i)	Neutron flux (fundamental mode) in state i
FVAR1,FVAR2, FVAR3,FVAR4	Free storage, not used in basic program
G(i)	Mean Collision rate per neutron in state i (γ_i in Eq. B6)
H(k,l,m)*	Probability that a neutron scattering from isotope m with initial energy in group k will end up in group l. (DEFAULT 0.0)
I,IA,IB,IC, ID, IE, IG,IH	General purpose indices
LEAK (n)	Leakage rate at time n
IND	n in Eq. 46
IPEAK	State number locating the peak of the neutron distribution
J,JJ	Do loop indices (unused in current version of MOD-5)
JROW	The number of rows in the matrix that have been defined
KROW	Number of the next row to be defined
L(j)**	Logical control variable (Appendix D)
LL	Temporary storage for NT

MAXR***	Maximum number of states in real matrix, including absorbing states (DEFAULT 74)
MAXT***	Maximum number of operating time steps allowed by dimensions in common block. (DEFAULT 400)
MAXV***	Maximum number of states in virtual matrix (including absorbing states) (DEFAULT 204)
N*	Number of energy states in real matrix, index of bottom energy state which is used to absorb all neutrons slowing down below lowest energy of interest (DEFAULT 71)
NC	An internally generated index for the external cross section file being read (READ, GET IT)
ND ***	Number of cross section sets that may be held in common (defined in READ, must be redefined if common block dimensions are changed. Value in existing program is ND=5)
NF*	Number of fissionable isotopes (DEFAULT 0)
NG(j)	Index of group to which state j (virtual index) belongs - if NG(j) = 0 state j straddles the boundary between two groups.
NGR***	Number of broad groups in cross section library (external file) (DEFAULT 26)
NI*	Number of isotopes in problem being executed (DEFAULT 1)
NII	Number of non-hydrogen isotopes (TRAN)
NIT	Number of time steps in interpolated output
NM	Number of active energy states in real matrix, $NM = N - 1$
NMO*	Number of time moments to be evaluated (DEFAULT 2)
NMOM(j)*	Listing of order of time moments to be evaluated (DEFAULT NMOM(1)=1, NMOM(2)=2)

NNG(k)	Index of lowest energy state lying completely within group k
NO	Index of external data set (cross sections) to be retrieved by GET IT
NOLIST(n)**	Index specifying the position of isotope n in the external data files. If NOLIST(1)=0 in primary data set (cards) the external data files (tape or disc) are not read.
NOMAX	Number of isotopes in external data file (tape or disc). This parameter is defined in READ (ftn) 14) and must be updated if the external data set is expanded or contracted.
NORM(i)	Normalization constant
NP1	N+1, Index of the absorbing state (real matrix) used for captured neutrons
NP2	N+2, Index of the absorbing state (real matrix) used for neutrons that have leaked from the system
NP3	N+3, Index of the absorbing state (real matrix) used for neutrons that have caused fission
NSCAT*	Used in connection with STAWID to define the number of lower energy states spanned by the distribution of scattered neutrons from a given state (Fig. 3) (DEFAULT 1)
NT	Maximum number of time steps allowed for problem in execution (must be \leq MAXT)
NTIME	Running index of time
NTITLE	Number of words allocated in storage for the problem title block.
NTM	NT - 1
NTOP	The number of virtual states above real state 1, - The true state number of real state i is $i + \text{NTOP}$

NVIR*	Number of energy states in virtual matrix - index of absorbing state for neutrons that have slowed below the energy range of interest (DEFAULT 71)
NVM	NVIR-1
N2N(m)	Yield of (n,2n) reactions in group 1 for isotope m.
OPCRIT*	Operating criteria - ratio of first operating time step to mean collision time at peak of source distribution when the matrix is generated (DEFAULT 0.4)
P(i,j)	Transition probability matrix (10-15) - the lower triangular portion of this array is used for storage of transfer coefficients in transposed form ($P_{i,j}$ in Eq. 15)
POP(i,1), POP(i,2)	Population vectors at times n and n+1 respectively - Defined over the range of the real matrix
PTP(i)*	Population vector for the complete state set (Virtual matrix) - an accounting vector used to keep track of population lost when the real matrix is shifted down the state structure. (DEFAULT 0.0)
R(j)	For some state i, the probability of the occurrence of at least j-1 collisions before a neutron leaves the state, $r_{i,j}$ (24)
RHO(m)**	Mass density of isotope m
RHON(m)	Nuclear density of isotope m times 10^{-24}
RI(j)	Resonance self shielding correction factors, 1-fission, 2-capture, 3-total, 4-elastic
RR(i)	Average number of collisions before a neutron leaves state i (24)

RSD(j)	Relative standard deviation of time distributions (58)
RSDX(j)	Relative standard deviation of time distributions calculated from asymptotic formulae
RSDR(j)	Percent difference of computed relative standard deviation with respect to asymptotic value
RSS(i,j,l,m)*	Resonance self shielding coefficients (ABN) (DEFAULT 1.0) i = group index j = index defining value of σ_0 1 - 0 barns 2 - 1 barn 3 - 10 barns 4 - 100 barns 5 - 1000 barns 6 - ≥ 10000 barns l = index specifying cross section type 1 - fission 2 - capture 3 - total 4 - elastic m = isotope index
SARG(n)	Resonance self shielding parameter $\sigma_{t,n}$ on page 44 of ABN (Ref. 6)
SCALE(k,m)*	Scale factor for isotope m in group k (21,26) (DEFAULT 1.0)
SCRIT*	Criteria for determining the fraction of an existing neutron distribution that is bypassed when the real matrix is shifted downward in energy (DEFAULT 0.001)
SDENS(n)	Slowing down density at time n and energy E(NVM)
SFLUX(i)	Steady state leakage flux in state i
SIGC(i,m) SIGE(i,m) SIGF(i,m)	

SIGN(i,m)	Capture, elastic scattering, fission, and inelastic scattering cross sections for isotope m in state i (divided by SIGT(i) after XSECT)
SIGT(i),SIGTR(i)	Total and transport cross sections in state i
SIGCB(k,m)** SIGEB(k,m) SIGFB(k,m) SIGNB(k,m) SIGTB(k,m)	Capture, elastic scattering, fission, inelastic scattering and total cross sections for isotope m in group k
SIGEP(m)	First factor on right side of Eq. 36 - for isotope m
SIGTBT(k) SIGTRB(k)	Total and transport cross sections in group k
SQCRIT*	Matrix squaring criteria (DEFAULT 0.2)
SSTAR(n)	Resonance self shielding parameter $\sigma_{t,n}^*$ on page 44 of ABN
SUMPOP	Total fraction of the population that has been bypassed by the real matrix as it moves down in energy
SZERO(1)	Resonance self shielding parameter $\sigma_{o,l}$ on page 42 of ABN
TIM(n)*	Time at time step n (DEFAULT TIM(1)=0.0)
TIMD(n)	$\frac{1}{2}(\text{TIM}(n) + \text{TIM}(n+1))$
TIMWID*	Width of source pulse in time (DEFAULT 0.0)
TITLE(k)**	Title block storage
TOTL(n)	Total neutron loss rate at time n
U(i)*	Upper lethargy boundary for state i (DEFAULT U(1)=0.0)

$V(i)$	Lower velocity boundary for state i
$VII(i)$	Inverse of expectation on inverse scattering velocity (as used in calculation of $B6$)
ZA,ZB,ZC,ZD,ZE	Free parameters

APPENDIX D

LOGICAL CONTROL VARIABLES

Note: The integer variable L provides 30 locations for variables which control the sequence of operations performed by MOD-5. L(1) through L(20) are reserved for the operations listed below. L(21) through L(30) remain unassigned and are available for the programmer's use. In general, if $L(n) = 1$ the operation listed is carried out and if $L(n) = 0$ the operation is not carried out. Note that L(2) must always be defined to be 1 in the problem data set and this fact is used in READ (ftn 81,82) as a simple consistency check on the punched cards.

n	IF(L(n).EQ.1)
1	Set up run only, execution terminates after GENTIM
2	Problem initialization variable.
3	The real array is at the bottom of the virtual array. L(3) = 1 initially if the real and virtual arrays have the same dimension, i.e., if $N = NVIR$.
4	Define the state structure from E(NVM) upward. Default (0) value assumes state structure is to be defined from E(1) downward.
5	On-line plot of distributions in lethargy at time determined in AMULT.
6	Call STAWID to optimize state structure.
7	Call EXACT to calculate and print asymptotic time moments.
8	The source distribution in energy and the value of TIM(1) are specified in the input data set. If TIM(1) is not specified a default value of 0.0 seconds is assumed.
9	The source is to be monoenergetic (located in state 2) and uniformly distributed in time for an interval given by the input value of TIMWID.
10	There is fissionable material in the system
11	The source has a delta function distribution in time and energy. All neutrons start in state 1.
12	Inelastic scattering occurs. This variable must have an input value of 0. If one or more rows of the variable H has non-zero elements the program redefines L(12) = 1.
13	IF(L(5).EQ.1) plot distributions in lethargy as with L(5) = 1 but at closer intervals as defined in AMULT.
14	Punch output
15	Print DENS and SDENS

<u>1</u>	<u>IF(L(n).EQ.1)</u>
16	Print FISS, LEAK, CAPT, and TOTL
17	Print LEAK, CAPT, TOTL, DENS and SDENS
18	Write out detailed distributions (Interpolated distributions are always written)
19	Disregard instructions to terminate on ROWSUM error (WRITE2). XSECT defines L(19) = 1 when (n,2n) reactions take place.
20	Internally generated call for MODPLOT
21-30	Unassigned

APPENDIX E

DESCRIPTIONS OF SUBROUTINE FUNCTIONS

The MOD-5 package consists of a control program, MAIN, and 20 subroutines. MAIN is discussed in detail in Section I of this document. The subroutines are briefly described below, Equations referenced are to be found in the paper in APPENDIX A.

- AMULT 1) Multiplies state vector into the transition matrix
 2) Evaluates fission, capture, leakage and total loss rate at time NT; and density and slowing down density in bottom energy group.
 3) Determines whether the lethargy dependent neutron density should be printed (punched). This is done if the current time step straddles a time 1×10^{-n} , 2×10^{-n} or 5×10^{-n} seconds where n is an integer. If $L(13) = 1$ the density in lethargy is also printed at times near 1.5, 3, and 7×10^{-n} seconds.
- EXACT (OPTIONAL ROUTINE) Calculates asymptotic values of positive or negative moments of DENS to any order using expressions quoted in Williams, The Slowing Down and Thermalization of Neutrons, Chapter IX, Section II. First and second positive moments of SDENS are calculated using results of Pal and Nemeth, Nukleonik, 1, 5, 165 (1959).
- GENRAT Generates new rows of the transition matrix using Eqs. 14 and 15, after testing to see which rows satisfy Eq. 44 for the time step in use.
- GENTIM 1) Determines the operating time step width from the values of OPCRIT and the state decay constant at the state with the greatest population.
 2) Determines generating time and parameters each time a new real transition probability array is generated. See Eqs. 44-46.
- GETIT Called from READ to retrieve isotope numbers, names and masses, and the cross section set, from external files. Data sets are read sequentially and discarded until the desired set is found.

INCOND	<ol style="list-style-type: none"> 1) Computes the average fission yield of neutrons in each state using fission yield parameters in Abagayan, et al. 2) Defines the source vector if it is to be a delta function or a fission spectrum. 3) Normalizes the source spectrum.
INTERP	Computes linearly interpolated values of time distributions (fission rate, etc.) on a uniform time mesh of up to 200 points. The first 20 points are spaced at $\frac{1}{4}$ the separation of the remaining points.
LIBRAR	This routine is intended for use with the ABN 26 group cross section library. 26 group microscopic cross sections are corrected for resonance self-shielding and converted to macroscopic cross sections. These are then used to define cross sections for each state with energy interval weighted averages for those states lying on a group boundary. The resonance self shielding coefficients are interpolated linearly if $\sigma_0 < 1$ and logarithmically if $\sigma_0 > 1$.
MODPLT	(OPTIONAL ROUTINE) Provides for on-line plotting of the lethargy dependent neutron density with no correction for non-uniformity of state widths.
MOMENT	<ol style="list-style-type: none"> 1) Computes up to 21 moments of time distributions. 2) Computes exponential extrapolation corrections to the time distributions. 3) Computes multiplication constant. 4) Computes first energy moment of steady state core and leakage fluxes.
NORMAL	Normalizes time dependent distributions and fluxes to unit integral over time or lethargy.
OUTPUT	<ol style="list-style-type: none"> 1) Computes relative standard deviation of time distributions. 2) Computes 26 group flux. 3) Prints or punches final output data.
PARAM	<ol style="list-style-type: none"> 1) Defines lethargy, energy, and velocity values for state boundaries; and lethargy, energy and velocity widths of each state. May start from pre-determined lethargies of state boundaries or lethargy widths of states.

- 2) Evaluates $NG(i)$ to define group to which state i belongs.
 - 3) Evaluates $NNG(k)$ to specify index of lowest energy state lying completely in group k .
- READ
- 1) Initializes parameters ND , $NOMAX$, $MAXT$, $MAXR$, $MAXV$, NGR , and $NTITLE$ which specify dimensionality of the common block and the number of isotopes in the external file.
 - 2) Assigns DEFAULT values.
 - 3) Re-orders isotopes in ascending mass order (Exception: when present, hydrogen is always listed last by READ).
 - 4) Reads all card input data.
 - 5) Provides parameter and vector initial values.
- SQUARE
- Squares the transition matrix. Elements of the newly squared matrix are stored in their transposed position (lower triangular) in the matrix until the squaring process has been completed. The elements are then transposed to their normal position. SQUARE provides two squaring routines, one for a matrix in which not all of the rows have been defined, and one for a fully defined matrix.
- STAWID
- (OPTIONAL ROUTINE) This routine is used to adjust the lethargy width of the states to obtain optimal treatment of elastic scattering. (see Appendix A, pp. 45-46). To employ this routine the user must define $NSCAT$, the number of lower energy states into which neutrons are allowed to scatter from a given state. STAWID automatically calculates an initial estimate of the desired lethargy width and then iterates on this to refine the value.
- TRAN
- This routine computes the transfer coefficient matrix (transition probabilities are assembled from these in GENRAT). The model assumes constant cross sections across each state. The transfer coefficient array is computed in transposed form so it may be stored in the lower triangular portion of the matrix $P(i,j)$. Transition probabilities are stored in the upper triangular and diagonal elements of the array. Inter-state inelastic scattering transfer probabilities are derived from the inelastic transfer matrices of the 26 group cross section set using energy interval weighting.
- WRITE1
- Lists input and computed data for problem description. If common block is altered such that ND (maximum number of isotopes in system) exceeds 5 this subroutine will have to be updated.

WRITE2 Lists state structure parameters and tests the transfer coefficient array to see that probabilities along a row (column in transposed form) sum to unity.

XSECT 1) Macroscopic cross sections are divided by the total cross section to normalize them for use in defining transfer coefficients.
 2) The inelastic scattering array is tested for (n,2n) reactions.
 3) The inelastic scattering array is normalized.

APPENDIX F
MOD-5 Listing


```

355 FLOOR(IA)=1.-EXP(-G(IA)*ZA))/G(IA)
360 DO 6 IA=1,NM
365   POP(IA,2)=PTP(IA+NTOP)
370   POP(NP1,2)=PTP(NVIR+1)
375   POP(NP2,2)=PTP(NVIR+2)
380   POP(NP3,2)=PTP(NVIR+3)
385   POP(N,2) REPRESENTS ONLY THE ACCUMULATION OF NEUTRONS SLOWING
390   DOWN BELOW THE LOWER BOUNDARY OF THE CURRENT REAL ARRAY.
395   POP(N,2)=0.
400   THE MAIN OPERATING LOOP (AN IMPLIED DO LOOP) BEGINS HERE.
405   CONTINUE
410   NI=NT+1
415   IF (L(9).EQ.0) GO TO 8
420   THE NEXT 4 STATEMENTS ARE EXECUTED WHEN THE SOURCE IS A PULSE OF
425   FINITE LENGTH. THIS PROGRAM AUTOMATICALLY GENERATES A SOURCE OF
430   DURATION=TIMWID AND DELTA FUNCTION SPECTRUM IN STATE 2 WHEN L(9)=1
435   IF (TIM(NT).LT.TIMWID) POP(2,2)=POP(2,2)+DT(NTIME)/TIMWID
440   IF (TIM(NT).LT.TIMWID) GO TO 8
445   POP(2,2)=POP(2,2)-(TIM(NT)+DT(NTIME)-TIMWID)/TIMWID
450   L(9)=0
455   CONTINUE
460   CALL AMULT
465   IF (L(5).EQ.1.AND.L(20).EQ.1) CALL MODPLT
470   L(20)=0
475   IF THE MAXIMUM NUMBER OF TIME STEPS (MAXT) HAS BEEN REACHED.
480   CONTROL IS PASSED TO FINAL DATA MANIPULATION AND OUTPUT ROUTINES.
485   IF (NT.EQ.MAXT) GO TO 16
490   THE TOTAL POPULATION OUTSIDE THE REAL ARRAY IS DETERMINED. IF
495   THIS EXCEEDS 99 PERCENT, CONTROL PASSES TO OUTPUT ROUTINES.
500   ACCUM=SUMPOP+POP(N,2)+POP(NP1,2)+POP(NP2,2)+POP(NP3,2)+PTP(NVIR)
505   ZX=ACCUM
510   IF (ACCUM.GT.99.AND.OFAS(NT).LT.00001) GO TO 16
515   IF THE REAL ARRAY LIES AT THE BOTTOM OF THE VIRTUAL ARRAY
520   EXECUTION CONTINUES WITHOUT FURTHER TESTING.
525   IF (L(3).EQ.1) GO TO 13
530   THE FRACTION OF THE POPULATION OF THE REAL ARRAY THAT HAS SLOWED
535   BELOW ITS LOWER ENERGY LIMIT IS EVALUATED. IF ZR EXCEEDS C.1
540   PERCENT A NEW REAL ARRAY WILL BE DEFINED. OTHERWISE THE NORMAL
545   CONTROL SEQUENCE CONTINUES.
550   ZR=POP(N,2)/(1.-ACCUM)
555   IF (ZR.LT.SCRIT) GO TO 13
560   THE NEXT ELEVEN STATEMENTS DETERMINE THE NEW POSITION OF THE REAL
565   ARRAY BY FINDING THE (VIRTUAL) STATE NUMBER SUCH THAT 0.1 PERCENT
570   OF THE EXISTING POPULATION WILL BE BYPASSED BY THE NEW ARRAY.
575   IR=0
580   ZC=0.
585   ZB=1.-ACCUM
590   DO 9 IA=2,N
595     ZC=ZC+POP(IA,2)
600     ZD=ZC/ZR
605     IF (ZD.LT.SCRIT) GO TO 9
610     IB=IA
615     GO TO 10
620   CONTINUE
625   IF=NTOP
630   NTOP IS THE NUMBER OF VIRTUAL STATES ABOVE THE REAL STATES.
635   THUS REAL STATE NUMBER 1 IS VIRTUAL STATE NUMBER NTOP+1.
640   WHEN THE FIRST STATES OF REAL AND VIRTUAL MATRICIES ARE CCINCIDENT
645   DEFINE NTOP = 0.
650   NTOP=NTOP+IR
655   THE VECTOR BTP IS UPDATED
660   DO 11 IA=1,NM
665     PTP(IA+1)=POP(IA,2)
670     PTP(NVIR)=PTP(NVIR)+POP(N,2)
675     PTP(NVIR+1)=POP(NP1,2)
680     PTP(NVIR+2)=POP(NP2,2)
685     PTP(NVIR+3)=POP(NP3,2)
690     SUMPOP=SUMPOP+ZC
695     IF THE LOWEST STATE OF THE NEW REAL ARRAY LIES BELOW THE LOWEST
700     STATE OF THE VIRTUAL ARRAY, THE REAL ARRAY IS REDUCED IN SIZE AND

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DO 4 IA=1,NM
4 FISS(NT)=FISS(NT)+POP(IA,1)*P(IA,NP3)*ANUNOR(IA)
5 CONTINUE
CAPT(NT)=POP(NP1,2)-POP(NP1,1)
LEAK(NT)=POP(NP2,2)-POP(NP2,1)
TOTL(NT)=LEAK(NT)+CAPT(NT)+POP(NP3,2)-POP(NP3,1)
DENS(NT)=POP(NM,1)
C*** THE FUNCTION SDENS(NT) IS VALID AFTER TIME STEP LL, WHEN THE REAL
C*** MATRIX REACHES THE BOTTOM OF THE VIRTUAL MATRIX.
SDENS(NT)=POP(N,2)-POP(N,1)
TIM(NT+1)=TIM(NT)+DT(NTIME)
DELT=DT(NTIME)/DT(1)
DO 6 IA=2,NM
6 IA=IA+1
IB=IA+NTOP
ZA=POP(IA,1)*FLCOR(IA)
SFLUX(IB)=SFLUX(IB)+ZA*P(IA,NP2)
FLUX(IB)=FLUX(IB)+ZA
C*** IF (NT.EQ.1) GO TO 7
C*** TEST TO SEE IF POPULATION VECTOR SHOULD BE PRINTED.
ZA=ALOG10(TIM(NT)-TIM(1))
ZB=AINI(ZA)
ZA=ZA-ZB+1.
ZB=ALOG10(TIM(NT)+DT(NTIME)-TIM(1))
ZC=AINI(ZB)
ZB=ZB-ZC+1.
IF (ZB.LT.ZA) ZB=ZB+1.
IF (ZA.LE.1. AND.ZB.GT.1.) GO TO 7
IF (ZA.LE..69897. AND.ZB.GT..69897) GO TO 7
IF (ZA.LE..30103. AND.ZB.GT..30103) GO TO 7
IF (L(L13).EQ.0) GO TO 12
IF (ZA.LE..17609. AND.ZB.GT..17609) GO TO 7
IF (ZA.LE..47712. AND.ZB.GT..47712) GO TO 7
IF (ZA.LE..84510. AND.ZB.GT..84510) GO TO 7
GO TO 12
7 CONTINUE
L(20)=1
C*** CONVERT POPULATION TO DENSITY IN LETHARGY AND WRITE DENSITY VECTOR
8 POP(IA,1)=POP(IA,1)/DU(IA+NTOP)
ID=NTOP+1
ZA=TIM(NT)-TIM(1)
WRITE(6,17) NT,TIM(1),ZA,ID
IC=10*((NTOP-1)/10)+11
ID=NTOP+1
IE=IC-ID
WRITE(6,14) ID,(POP(IA,1),IA=1,IE)
IC=((NM+NTOP)/10)*10+1
IG=(IE-IC)/10
DO 9 IB=1,IG
IC=IC+9-NTOP
IH=IC-NTOP
WRITE(6,14) (IC,POP(IA,1),IA=IH,IE)
9 IC=IC+10
IC=IC-NIOP
WRITE(6,14) (IC,POP(IA,1),IA=ID,NM)
WRITE(6,13) (POP(IA,1),IA=N,NP3)
C*** CONVERT DENSITY BACK TO POPULATION
DO 10 IA=2,NM
10 POP(IA,1)=POP(IA,1)*DU(IA+NTOP)
C*** COMPUTE ENERGY AND VELOCITY MOMENTS OF THE POPULATION VECTOR.
ZA=0.
ZB=0.
ZC=0.
ZE=0.
DO 11 IA=1,NM
11 IA=IA+NTOP
ZA=ZA+POP(IA,1)
ZB=ZB+POP(IA,1)*EII(IAP)
ZC=ZC+POP(IA,1)*EII(IAP)*EII(IAP)

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160 ZA=ALPHA(1)
165 VSIG=VII(NVM)*SIGEB(25,1)
170 XI=1
175 IF (ZA.GT.0.) XI=1.+(ZA*ALOG(ZA))/(1.-ZA)
180 CMOMENTS OF DENS
185 DO 4 IA=1,NMC
190 IF (NMOM(IA).LT.0) GO TO 2
195 POSITIVE MOMENTS
200 ID=NMOM(IA)
205 EXM(IA,2)=(1.+C6/VSIG)**NMOM(IA)
210 DO 1 IB=1,ID
215 1 EXM(IA,2)=EXM(IA,2)*(ZB/(1.-(2.*(1.-ZA**((1.+ZB/2.))))/((ZB+2.)*(1.-
220 (ZA))))
225 GC TO 4
230 IF (NMOM(IA,2).EQ.0)
235 IF (NMOM(IA,2).EQ.1) GO TO 4
240 IF (NMOM(IA,2).EQ.(-1)) GO TO 4
245 ID=-NMOM(IA)-1
250 DO 3 IB=1,ID
255 1 EXM(IA,2)=EXM(IA,2)*(VSIG/1.E06)*(1.-(2.*(1.-ZA**((1.-
260 (ZB/2.))))/((2.-ZB)*(1.-ZA))))/(-ZB)
265 IF (IB.EQ.2) EXM(IA,2)=EXM(IA,2)*(VSIG/1.E06)*(1.-ZA**((1.-
270 (ZB/2.))))/((2.-ZB)*(1.-ZA))))/(-ZB)
275 IF (IB.EQ.2) EXM(IA,2)=EXM(IA,2)*(VSIG/1.E06)*(1.-ZA**((1.-ZA)
280 1)))/(-2.)
285 3 CONTINUE
290 4 CONTINUE
295 CMOMENTS OF SPENS
300 ZD=0.
305 ZB=1.
310 IF (ZA.GT.0.) ZB=1.+(ZA*ALOG(ZA))/(1.-ZA)
315 ZE=V(1)*SIGEB(25,1)
320 ZF=V(NVM)*SIGEB(25,1)
325 ZG=V(NVM)/V(1)
330 IF (ZA.GT.0.) ZD=(.5*ZB*ZG)/(1.+ALOG(ZA)/(1.-ZA))
335 EXM(1,1)=(2.*(1.-.5*ZB*ZG*AMASS(1)))/(ZB*ZF)
340 EXM(2,1)=(2.*(1.-.5*ZB*ZG*AMASS(1)))/(ZB*ZF)
345 1 S(1)*ZG*(1.+ZD))/(ZB*ZF*ZF)
350 ZA=EXM(1,1)*1.E06
355 ZB=EXM(2,1)*1.E12
360 EXM(1,1)=0.
365 EXM(2,1)=0.
370 DO 5 IA=1,NMO
375 IF (NMOM(IA).EQ.1) EXM(IA,1)=ZA
380 IF (NMOM(IA).EQ.2) EXM(IA,1)=ZB
385 DO 5 IB=1,2
390 5 IF (EXM(IA,IB).GT.0.) ERR(IA,IB)=CORMOM(IA,IB+4)/EXM(IA,IB)-1.
395 RETURN
400 END
405

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**** GENRAT ****

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5 SUBROUTINE GENRAT
10 COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SFUX(200),RR(200),G(
15 171),SIGT(71),SIGF(71),SIGE(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
20 2,DB2(71),SIGTR(71),FISS(400),LEAK(400),CAPT(400),TOTL(400),SCENS(4
25 300),DENS(400),FLCOR(200),TIM(400),U(201),DU(201),V(201),CV(201),E(
30 4201),DE(201),E1(201),V1(201),ANUNDR(71),NG(201),FVAR1(204),FVAR2
35 5(204),FVAR3(204),FVAR4(204)
40 COMMON ALP(5),ALPHA(5),AMASS(5),AMOM(21,6),ATOM(10),BGF(26),BGSF(2
45 16),CURMOM(21,6),DEABN(26),DT(21),EABN(26),EMIN(5),ERR(21,2),EXM(21
50 2,2),H(12,12,5),L(30),NMOM(21),NNG(26),NORM(6),RHON(5),R(6),RHO(5),
55 3RI(4),PSS(26,6,4,5),SARG(5),SIGTB(26,5),SIGFB(26,5),SIGNE(26,5),SI
60 4GCB(26,5),SIGEB(26,5),AMUB(26,5),DB2B(26),SIGTRB(26),SIGTR(26),SI
65 5GEB(26,5),SIGTAR(5),SCALE(26,5),TTT(26,5),ALAM(5),RSCD(6),RSDR

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170 ZA=PTP(2)
175 IF (ZA.EQ.0.) GO TO 2
180 DO 1 IA=3,NM
185 IF (PTP(IA).LT.ZA) GO TO 1
190 ZA=PTP(IA)
195 IPEAK=IA
200 CONTINUE
205 1 DEFINE FIRST OPERATING TIME STEP.
210 2 DT(1)=OPCRIT/G(IPEAK-NTOP)
215 3 GENERATING TIME ROUTINE.
220 4 INITIAL ESTIMATE OF GENERATING TIME.
225 5 ZA=CRIT/G(IPEAK-NTOP)
230 6 ZB=(ALOG10(DT(NTIME)/ZA)/ALOG10(2.))
235 7 GENERATING INDEX (N IN EQ. 46)
240 8 IND=N(ZB)
245 9 FINAL ESTIMATE OF GENERATING TIME.
250 10 DCT=DT(NTIME)/(2.**IND)
255 11 JROW=0
260 12 KRCW=1
265 13 IF (L(11).EQ.0) JROW=1
270 14 IF (L(11).EQ.0) KROW=2
275 15 IF (L(11).EQ.1) WRITE (6,4) IND
280 16 IF (IND.GT.1) WRITE (6,5) IND,DDT,IPEAK,G(IPEAK-NTOP)
285 17 RETURN
290 18
295 19 4 FORMAT ('0',////////,' GENERATING INDEX =',I3,' AND ROW GENERATIO
300 20 IN ROUTINE HAS BEEN BYPASSED.',////////)
305 21 5 IN FORMAT ('0',////////,' GENERATING ROUTINE IS BEING CALLED FOR',I3,' I
310 22 ITERATIONS BEGINNING AT TIME DDT =',IPE10.3,' SECONDS.',/,,' DCT IS
315 23 2 DEFINED FOR VIRTUAL STATE',I4,' FOR WHICH THE DECAY CONSTANT IS',I
320 24 3PE10.3,'.',/,////////)
325 25 END

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**** GETIT (****

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SUBROUTINE GETIT (NO,NC,LASTNO)
COMMON P(74,74),POP(74,2),PTPL(204),FLUX(200),SFLUX(200),RR(200),G(
171),SIGT(71),SIGF(71,5),SIGN(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
2,CB2(71),SIGTR(71),FISS(400),LEAK(400),CAPT(400),TOIL(400),SDENS(4
300),DENS(400),FLCOR(200),TIM(400),U(201),V(201),DV(201),E(
51204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),AMOM(21,6),ATOM(10),BGF(26),EGSF(2
16),COM MOM(21,6),DEABN(26),DT(21),EABN(26),EMIN(5),ERR(21,2),EXM(21
2),H(12,12,5),L(30),NMOM(21),NNG(26),NORM(6),RHON(5),RHO(5),SI
3RT(4),RSS(26,6,4,5),SARG(5),SIGTB(26,5),SIGFBI(26,5),SIGNB(26,5),SI
4GCB(26,5),SIGEB(26,5),AMUB(26,5),DB2B(26),SIGIRB(26),SIGIBI(26),SI
5GEP(5),SSSTAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),RSD(6),RSCR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DDT,DTI,EFFK,ESTK,I,I
1A,IB,IC,ID,IE,IF,IH,IND,IPEAK,J,JJ,JRCW,KROW,LL,MAXR,MAXT,MAXV,N,N
2A,ND,NCR,NF,NI,NIT,NM,NMO,NP1,NP2,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
3OP,NVIR,NVM,OPCRIT,SCRIT,SQCRIT,SUMPOP,TIMMID,ZA,ZB,ZC,ZD,ZE
REAL LEAK,NORM
DIMENSION DTIM(400),TIMD(400),DINT(200,6),TINT(200),DIST(400,6)
1 EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
2 (FISS,DIST)
3 COMMON NUM1(5),NUM2(5),NOMAX
4 INTEGER NUM2*4
THIS SUBROUTINE RETRIEVES ISOTOPE AND CROSS SECTION DATA FROM
EXTERNAL FILES AND PLACES IT IN APPROPRIATE LOCATIONS.
DATA SETS ARE READ SEQUENTIALLY AND DISCARDED UNTIL THE DESIRED
SET IS FOUND.
ND = INDEX NUMBER OF DATA SET TO BE RETRIEVED. BETWEEN 1 AND NOMAX

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IF (ND.GT.LASTNO) GO TO 1
LASTNO=0
REWIND 1
1 IB=LASTNO+1
DO 2 IC=IB,NC
  LASTNO=LASTNO+1
2 READ (1,3) NUM2(NC),ATOM(2*NC-1),ATOM(2*NC),AMASS(NC), (SIGTB(IC,NC
  1),IC=1,NGR), (SIGFB(IC,NC),IC=1,NGR), (SIGNB(IC,NC),IC=1,NGR), (SIGCR
  2(IC,NC),IC=1,NGR), (SIGEB(IC,NC),IC=1,NGR), (AMUB(IC,NC),IC=1,NGR), (
  3(H(IC,IC,NC),ID=1,12), (RSS(IC,IC,IC,IC,IC=1,NGR),ID=1,
  46),IE=1,4)
RETURN
C*** 3 FORMAT (4(232A4))
      END

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**** INCOND ****

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SUBROUTINE INCOND
COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SFLUX(200),RR(200),G(
  171),SIGT(71),SIGF(71,5),SIGN(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
2,DB2(71),SIGTR(71),FISS(400),LEAK(400),CAPT(400),TOTL(400),SDENS(4
  300),DENS(400),FLCOR(200),TIM(400),U(201),DU(201),V(201),DVI(201),E(
  4201),DE(201),EII(201),VII(201),ANUNOR(71),NG(201),FVAR1(204),FVAR2
  5(204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),AMOM(21,6),ATOM(10),BGF(26),PGSF(2
  16),CORMOM(21,6),DEABN(26),DT(21),EABN(26),ERR(21,2),EXM(21
  2,2),H(12,12,5),L(30),NMOM(21),NNG(26),NORM(6),RHON(5),RHO(5),
  3RI(4),RSS(26,6,4,5),SARG(5),SIGTB(26,5),SIGFB(26,5),SIGNB(26,5),SI
  4GCB(26,5),SIGEB(26,5),AMUB(26,5),DB2B(26),SIGTRB(26),SIGTB(26),SI
  5GEP(5),SSSTAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),RSDR(6),RSDR
  6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DDT,DTT,EFFK,ESVK,I,I
  1A,IB,IC,ID,IE,IG,IH,IND,IPEAK,J,JJ,JRCW,KROW,LL,MAXR,MAXT,MAXV,N,N
  2A,ND,NGR,NF,NH,NIT,NM,NMO,NP1,NP2,NP3,NSCAT,NT,NTIME,NTITLE,ATM,NT
  3OP,NVIR,NVM,OPCRIT,SCRIT,SQCRT,SUMPOP,TIMWID,ZA,ZB,ZC,ZD,ZE
REAL LEAK,NORM
DIMENSION DTIM(400),TIMD(400),DINT(200,6),TINT(200),DIST(400,6)
1)
EQUIVALENCE (P,DIIM), (P(401),TIMD), (P(801),DINT), (P(3201),IINT)
I, (FISS,DIST)
INCOND DEFINES AND NORMALIZES THE INITIAL STATE VECTOR AND
COMPUTES NEUTRON YIELDS PER FISSION IN EACH STATE (ANUNOR).
C***
C***
C***
IF (L(10).EQ.0) GO TO 18
ZA=0.
ZB=0.
DO 1 IB=1,NM
  1 ANUNOR(IB)=0.
  DO 15 IA=1,NI
    IC=AMASS(IA)-231.
    CHECK TO SEE IF ISOTOPE IS FISSIONABLE.
    IF (IC.LT.1) GO TO 15
    IF (IC.GT.11) GO TO 15
    GO TO (2,3,4,5,6,7,8,9,10,11,12), JC
  2 ZA=1.95
    ZB=.14E-06
    GO TO 13
  3 ZA=2.49
    ZB=.13E-06
    GO TO 13
  4 ZA=2.37
    ZB=.13E-06
    GO TO 13

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5 ZA=2.42
  78=.135E-06
  GO TO 13
6 ZA=2.38
  78=.135E-06
  THERE IS NO FISSION ISOTOPE OF MASS 237.
  7 GC TO 13
8 ZA=2.4
  78=.14E-06
  GO TO 13
9 ZA=2.87
  78=.13E-06
  GO TO 13
10 ZA=2.8
  78=.13E-06
  GO TO 13
11 ZA=2.96
  78=.135E-06
  GO TO 13
12 ZA=2.85
  78=.135E-06
  GO TO 14
13 DO 14 18=1, NM
  C*** DEFINES THE AVERAGE NEUTRON YIELD PER FISSION IN EACH STATE.
  C*** A FISSION RATE WEIGHTED AVERAGE OVER THE ISOTOPE PRESENT.
14 ANUNOR(18)=ANUNOR(18)+SIGF(18,IA)*(ZA+Z8*E11(18+NTOP1))
15 CONTINUE
  DO 17 1A=1, NM
  ZA=0.
  CC 16 18=1, NI
  16 ZA=ZA+SIGF(1A, 18)
  IF (ZA.NE.0) ANUNOR(1A)=ANUNOR(1A)/ZA
  17 CONTINUE
  18 CONTINUE
  IF (L(2).EQ.0) GO TO 25
  IF (L(9).EQ.1) GO TO 25
  ZD=0.
  IF (L(11).EQ.1) GO TO 20
  IF (L(18).EQ.1) GO TO 21
  C*** DEFINES SOURCE VECTOR ACCORDING TO THE WATT FISSION SPECTRUM.
  DO 19 1A=2, NVN
  ZA=(E(1A-1)+E(1A))*6000000
  ZB=-ZA/0.965
  ZC=SQR(2.29*ZA)
  PTP(1A)=(EXP(ZB))*SINH(ZC)*DE(1A)
  19 ZC=ZD+PTP(1A)
  ZD=L/ZD
  GO TO 23
  C*** DELTA FUNCTION SOURCE
  20 PTP(1)=1.
  GO TO 23
  21 ZC=0.
  DO 22 1A=2, NVN
  22 ZD=ZD+PTP(1A)
  IF (ZD.GT.0.) ZD=L/ZD
  23 CONTINUE
  C*** NORMALIZE SOURCE VECTOR
  DO 24 1A=2, NVN
  24 PTP(1A)=PTP(1A)*ZD
  WRITE (6,26)
  WRITE (6,27) (PTP(1A), 1A=1, NVN)
  25 CONTINUE
  RETURN
  C***
  C***
26 FORMAT ('1', '/', '/', '1', '-----SOURCE VECTOR POP(1,1) (NEUTRONS PER STATE
1)-----', '/')
27 FORMAT ('1P10E10.3)
END

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SUBROUTINE INTERP
COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SFLUX(200),RR(200),G(
171),SIGT(71),SIGF(71,5),SIGN(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
2,DB2(71),SIGR(71),FISS(400),LEAK(400),CAPT(400),TOTL(400),SCENS(74
300),DENS(400),FLCUR(200),TIM(400),U(201),DU(201),V(201),DV(201),E(
4201),DE(201),EII(201),VII(201),ANUNOR(71),NGI(201),FVAR1(204),FVAR2
5(204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),AMOM(21,6),ATOM(10),BGF(26),BGSF(2
16),GORMGM(21,6),DEABN(26),DT(21),EABN(26),EMIN(5),ERR(21,2),EXM(21
2,2),H(12,12,5),L(30),NMOM(21),NNG(26),NORM(6),RHON(5),R(6),RFO(5),
3RI(4),RSS(26,6,4,5),SARG(5),SIGTB(26,5),SIGFEB(26,5),SIGNB(26,5),S
4GCB(26,5),SIGEB(26,5),AMUB(26,5),DB2B(26),SIGIRB(26),SIGTBT(26),S
5GEP(5),SSAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),RSC(6),RSCR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DDT,DTI,EFFK,ESTK,I,1
1A,IB,IC,ID,IF,IG,IH,IND,IPEAK,J,JJ,JRCW,KROW,LL,MAXR,MAXV,N,N
2A,NO,NGR,NF,NINIT,NM,NMG,NPI,MP2,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
3OP,NVIR,NVM,OPCRIT,SCRIT,SQCRIT,SUMPOP,TIMWID,ZA,ZB,ZC,ZD,ZE
REAL LEAK,NORM
DIMENSION CTIM(400),TIMD(400),DINT(200,6),TINT(200),DIST(400,6
1)
EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
1, (FISS,DIST)
C***
C*** INTERP COMPUTES LINEARLY INTERPOLATED VALUES OF TIME DISTRIBUTIONS
C*** ON A UNIFORM TIME MESH OF 200 POINTS. THE FIRST 20 POINTS ARE
C*** SPACED AT 1/4 THE SEPARATION OF THE REMAINING POINTS.
C***
DO 1 IA=1,6
DO 1 IB=1,200
DINT(IB,IA)=0
C*** CALCULATE TIME STEP WIDTH.
Z=(TIM(NT)-TIM(1))/200.
ZA=ALOG10(Z)
ZB=AIN1(ZA)
ZA=10.***(ZA-ZB+1.)
DTI=10.
IF (ZA.LE.5.) DTI=5.
IF (ZA.LE.2.) DTI=2.
DTI=DTI*(10.***(ZB-1.))
C*** CALCULATE THE NUMBER OF ITERATIONS REQUIRED
NIT=INT((TIM(NT)-TIM(1))/DTI)+15
IF (NIT.GT.200) DTI=DTI*2.
IF (NIT.LT.65) DTI=DTI/2.
NIT=INT((TIM(NT)-TIM(1))/DTI)+15
DO 2 IA=1,20
ZA=IA
2 TINT(IA)=ZA*DTI*.25+TIM(1)
DO 3 IA=21,NIT
ZA=IA-15
3 TINT(IA)=ZA*DTI+TIM(1)
IF (L(16).EQ.1) GO TO 6
INTERPOLATION OF DENSITY
NB=1
DO 5 IA=1,NIT
NBEG=NB
DO 4 IB=NBEG,NTM
IF (TIM(IB+1).LT.TINT(IA)) GO TO 4
DINT(IA,6)=DENS(IB)+(DENS(IB+1)-DENS(IB))*(TINT(IA)-TIM(IB))/(TIM(
118+1)-TIM(18))
NB=IB-1
GO TO 5
4 CONTINUE
5 CONTINUE
6 CONTINUE
C*** INTERPOLATION OF ABSORBING STATE DISTRIBUTIONS
IC=1

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240 IF (SARG(IB).GT.1.) IZ=INT(ALOG10(SARG(IB)))+2
245 IF (SARG(IB).GT.1.0E05) IZ=6
250 IF (SARG(IB).GT.1.0E05) NFLAG=1
255 IF (NFLAG.EQ.1) WRITE (6,17) IB, IA
260 ZA=IZ+2
265 DO 3 IC=1,4
270 RI(IC)=RSS(IA,2,IC,IB)+SARG(IB)*(RSS(IA,2,IC,IB)-RSS(IA,1,IC,IB))
275 IF (IZ.GT.1) RI(IC)=RSS(IA,IZ,IC,IB)+(RSS(IA,IZ+1,IC,IB)-RSS(IA,IZ
280 1,IC,IB))*(ALOG10(SARG(IB))+ZA)
285 3 CONTINUE
290 C*** IF IZ=1 LINEAR INTERPOLATION, OTHERWISE LOGARITHMIC.
295 4 SSTAR(IB)=SIGNB(IA,IB)+SIGFB(IA,IB)*RI(1)+SIGCB(IA,IB)*RI(2)+SIGEB
300 1(IA,IB)*RI(4)
305 SUM=0.
310 DO 5 IR=1,N1
315 SLM=SUM+RHON(IB)*SSTAR(IB)
320 DO 7 IB=1,N1
325 SZERO(IP)=SUM/RHON(IB)-SSTAR(IB)
330 IZ=1
335 IF (SZERO(IB).GT.1) IZ=INT(ALOG10(SZERO(IB)))+2
340 ZA=IZ+2
345 DO 6 IC=1,4
350 RI(IC)=RSS(IA,2,IC,IB)+SZERO(IB)*(RSS(IA,2,IC,IB)-RSS(IA,1,IC,IB))
355 IF (IZ.GT.1) RI(IC)=RSS(IA,IZ,IC,IB)+(RSS(IA,IZ+1,IC,IB)-RSS(IA,IZ
360 1,IC,IB))*(ALOG10(SZERO(IB))+ZA)
365 1,IC,IB)
370 6 CONTINUE
375 SIGEB(IA,IB)=SIGFB(IA,IB)*RI(1)*RHON(IB)
380 SIGCB(IA,IB)=SIGCB(IA,IB)*RI(2)*RHON(IB)
385 SIGFB(IA,IB)=SIGFB(IA,IB)*RI(3)*RHON(IB)
390 SIGEB(IA,IB)=SIGEB(IA,IB)*RI(4)*RHON(IB)
395 SIGNB(IA,IB)=SIGNB(IA,IB)*RHON(IB)
400 7 CONTINUE
405 SIGTRB(IA)=0.
410 DO 8 IB=1,N1
415 SIGTRB(IA)=SIGTRB(IA)+SIGTB(IA,IB)-AMUB(IA,IB)*SIGEB(IA,IB)
420 DB2B(IA)=BUCKLE/13.*SIGTRB(IA)-((BUCKLE/SIGTRB(IA))*2)/5.
425 SIGTB(IA)=DB2B(IA)
430 DO 9 IR=1,N1
435 SIGBT(IA)=SIGBT(IA)+SIGEB(IA,IB)+SIGCB(IA,IB)+SIGNB(IA,IB)+SCALE
440 1(IA,IB)*(1.+2./13.*AMASS(IB))-AMUB(IA,IB)*SIGEB(IA,IB)
445 10 CONTINUE
450 C*** IN THE REMAINING STATEMENTS CROSS SECTIONS ARE DEFINED FOR STATES
455 C*** OF THE NEW REAL ARRAY
460 11 DO 16 IA=1,NM
465 IZ=NG(IA+NTOP)
470 IF (IZ.EQ.0) GC TO 13
475 IF REAL STATE IA (VIRTUAL STATE IA+NTOP) DOES NOT LIE ON A BROAD
480 GROUP BOUNDARY THE NEXT NINE STATEMENTS ARE EXECUTED.
485 DO 12 IB=1,N1
490 AMUB(IA,IB)=AMUB(IZ,IB)
495 SIGC(IA,IB)=SIGCB(IZ,IB)
500 SIGN(IA,IB)=SIGNB(IZ,IB)
505 SIGF(IA,IB)=SIGFB(IZ,IB)
510 SIGE(IA,IB)=SIGEB(IZ,IB)
515 SIGT(IA)=SIGTB(IZ)
520 DB2(IA)=DB2B(IZ)
525 SIGTR(IA)=SIGTRB(IZ)
530 GO TO 16
535 IF REAL STATE IA LIES ON A GROUP BOUNDARY ENERGY WEIGHTED AVERAGE
540 CROSS SECTIONS ARE COMPUTED.
545 13 IV=IA+NTOP-1
550 W1=(E(IV)-EABN(NG(IV)))/(E(IV)-E(IV+1))
555 W2=(EABN(NG(IV))-E(IV+1))/(E(IV)-E(IV+1))
560 IV=NG(IV)
565 DO 14 IB=1,N1
570 AMUB(IA,IB)=W1*AMUB(IV,IB)+W2*AMUB(IV+1,IB)
575 SIGC(IA,IB)=W1*SIGCB(IV,IB)+W2*SIGCB(IV+1,IB)
580 SIGN(IA,IB)=W1*SIGNB(IV,IB)+W2*SIGNB(IV+1,IB)

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14 SIGF(IA,IB)=W1*SIGFB(IV,IB)+W2*SIGFR(IV+1,IB)
   DB2(IA)=W1*DB28(IV)+W2*DB28(IV+1)
   SIGTR(IA)=W1*SIGTRB(IV)+W2*SIGTRB(IV+1)
   SIGT(IA)=DB2(IA)
   DO 15 IB=1,N
   15 SIGT(IA)=SIGT(IA)+SIGC(IA,IB)+SIGN(IA,IB)+SCALE(IV,IB)*(1.+2./(3.*
   1AMASS(IV))-AMU(IA,IB))*SIGE(IA,IB)+SIGF(IA,IB)
16 CONTINUE
16 RETURN
C***
17 FORMAT ('0',***** CAUTION *****,//,'SIGMAZERO EXCEEDS 1.0E 05 FOR
   1ISCTOPE NUMBER',I2,' IN GROUP NUMBER',I3,'. THIS LIES OUTSIDE THE
   2BOUNDS OF,/,THE RESONANT SELFSHIELDING TABLE THAT ACCOMPANIES I
   3THIS PROGRAM. A DEFAULT VALUE OF 1.0E 04 IS ASSUMED,/, BUT THE US
   4ER SHOULD REFER TO THE ARN 26 GROUP LISTING TO SEE IF THIS ASSUMPT
   5I/N IS ACCEPTABLE.,,//')
   END

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**** MODPLT ****

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SUBROUTINE MODPLT
COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SELUX(200),RR(200),G(
171),SIGT(71),SIGF(71,5),SIGN(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
2,CB2(71),SIGTR(71),FISS(400),LEAK(400),CAPT(400),TOTL(400),SCENS(4
300),DENS(400),FLCOR(200),TIM(400),U(201),DU(201),DV(201),E(
4201),DE(201),EIT(201),V(201),ANUNDR(71),NG(201),FVAR1(204),FVAR2
5(204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),AMOM(21,6),ATOM(10),BGF(26),EGSF(2
16),CORMOM(21,6),DEARN(26),DT(21),EABN(26),EMIN(5),ERR(21,2),EXM(21
2,2),H(12,12,5),L(30),NMOM(21),NNG(26),NORM(6),RHON(5),R(6),RHO(5),
3RI(4),RSS(26,6,4,5),SARG(5),SIGFB(26,5),SIGFB(26,5),SIGNB(26,5),SI
4GCB(26,5),SIGEB(26,5),AMUB(26,5),DB2B(26),SIGTRB(26),SIGTR(26),SI
5GEP(5),SSSTAR(5),SZERO(5),SCALE(26,5),JITLE(72),ALAM(5),RSC(6),RSDR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DDT,DTI,EFFK,ESTK,I,I
IA,IB,IC,ID,IE,IG,IH,IND,IPEAK,J,JJ,JRCM,KROW,LL,MAXR,MAXI,MAXV,N,N
2A,ND,NGR,NF,NI,NIT,NM,NMO,NP1,NP2,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
3OP,NVIR,NVM,OPCRIT,SCRIT,SQCRIT,SUMPOP,TIMWID,ZA,ZB,ZC,ZD,ZE
REAL LEAK,NORM
DIMENSION DTIM(400),TIMD(400),DINT(200,6),TINT(200),DIST(400,6
1),EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
1, (FISS,DIST)
C***
C***
C***
C***
MODPLT PROVIDES A ROUGH PLOT OF THE LETHARGY DEPENDENT NEUTRON
DENSITY, ONE POINT FOR EACH STATE IN REAL VECTOR WITH NO
CORRECTION FOR NON-UNIFORMITY OF STATE WIDTHS.
DIMENSION M(103),MALL(3),SC(11),ISYM(21),MPI(103),MRP(103)
DATA ISYM/14*,14/,IP,IM/IH,IH-/
DATA I1/IH1/
REDEFINE FUNCTION AND DETERMINE RANGE.
YMAX=1.
YMIN=0.
TOP BORDER
DO 1 IA=1,103
1 M(IA)=IM
2 M(IA)=IP
DELTA=(YMAX-YMIN)*.1
SC(1)=YMIN
DO 3 IA=2,11
3 SC(IA)=SC(IA-1)+DELTA
WRITE (6,9) SC
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100 DIMENSION DTIM(400), TIMD(400), DINT(200,6), TINT(200), DIST(400,6)
105
110 1) EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
115 1, (FISS,DIST)
120
125 MOMENT COMPUTES: (1) UP TO 21 MOMENTS OF TIME DEPENDENT DISRIB-
130 UTIONS; (2) EXPONENTIAL EXTRAPOLATION CORRECTIONS TO THE TIME
135 MOMENTS; (3) MULTIPLICATION CONSTANT, AND (4) FIRST ENERGY MOMENTS
140 OF STEADY STATE CORE AND LEAKAGE SPECTRA.
145
150 IPEAK=1
155 CLEAR TRANSITION MATRIX FOR STORAGE OF NEW VARIABLES.
160 DO 1 IA=1,MAXR
165 DO 1 IB=1,MAXR
170 P(IA,IB)=0.
175 TIMD IS DEFINED AT MID-STEP FOR ABSORBING TRANSITIONS.
180 DO 2 IA=1,NTM
185 TIMD(IA)=.5*(TIM(IA+1)+TIM(IA))
190 TIMD(NT)=TIMD(NTM)*2.-TIMD(NTM-1)
195 IF (L(16).EQ.1) GO TO 10
200 TIME STEP WIDTHS
205 DO 3 IA=2,NTM
210 TIM(IA)=TIMD(IA+1)-TIMD(IA)
215 MOMENTS OF NEUTRON DENSITY IN STATE NVM. DENS IS TEMPORARILY
220 RENORMALIZED TO AN AMPLITUDE THAT MINIMIZES TRUNCATION AND ROUND-
225 OFF ERRORS AND UNDERFLOW OR OVERFLOW ERRORS.
230 FIND PEAK OF DENSITY
235 DO 4 IA=20,NTM
240 IF (DENS(IA+1).LT.DENS(IA)) GO TO 5
245 CONTINUE
250 IPEAK=IA
255 RENORMALIZE TO UNIT TIME AT PEAK
260 ZA=TIM(IPEAK)
265 DO 6 IA=1,NT
270 TIM(IA)=TIM(IA)/ZA
275 DENS(IA)=DENS(IA)*ZA
280 ID=5
285 IF (ID=5)
290 COMPUTE FIRST ESTIMATE OF MOMENTS (AMOM)
295 DO 8 IA=1,NMO
300 ZD=0.
305 IC=NMO*(IA)
310 DO 7 IB=2,NTM
315 ZC=ZD+DENS(IB)*DTIM(IB)*(TIM(IB)**IC)
320 AMOM(IA,6)=ZC
325 DO 9 IA=1,N
330 TIM(IA)=TIM(IA)*ZA
335 MOMENTS OF REMAINING TIME DEPENDENT DISTRIBUTIONS.
340 DO 10 CONTINUE
345 TIME STEP WIDTHS
350 DO 11 IA=1,NTM
355 DTIM(IA)=TIM(IA+1)-TIM(IA)
360 TIM(NT)=TIM(NTM)
365 RENORMALIZE TO UNIT TIME = SORT OF PRODUCT OF SMALLST AND LARGST
370 VALUES OF TIMD
375 ZC=SQRT(TIMD(2)*TIMD(NTM))
380 IF (ID=5)
385 ID=5
390 IF (L(17).EQ.1) ID=2
395 IF (L(16).EQ.1) ID=1
400 IF (L(16).EQ.1) ID=4
405 IF ONLY DENS IS RENORMALIZED UNIT TIME = TIME OF PEAK
410 IF (ID=5.AND.ID=4) ZC=TIMD(IPEAK)
415 GO 12 ID=1,IF
420 DO 12 IB=1,NT
425 DIST(IB,IA)=DIST(IB,IA)*ZC
430 ZC=1./ZC
435 DO 13 IA=1,NT
440 TIMD(IA)=TIMD(IA)*ZC
445

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C***
14  CALCULATE MOMENTS
    DO 14 IA=1,NMO
      IC=NMOM(IA)
      DO 14 IB=2,NTM
        ZC=DTIM(IB)*(TIMD(IB)**IG)
      DD 14 IC=ID,IE
    AMOM(IA,IC)=AMOM(IA,IC)+DIST(IB,IC)*ZD
    IF (L(16).EQ.0) IE=6
  CALCULATE EXTRAPOLATION CORRECTIONS
  DO 18 IA=ID,IE
  DO 15 IR=1,NMO
    CORMOM(IR,IA)=AMOM(IR,IA)
    IF (DIST(NT-2,IA).LE.0.) GO TO 18
    Z1=DIST(NT-6,IA)/DIST(NT-2,IA)
    Z2=(ALOG(Z1)*Z1)/(TIM(NT-2)-TIM(NT-6))
    IF (IA.LE.5) Z2=(ALOG(Z1))/(TIMD(NT-2)-TIMD(NT-6))
    Z3=1.+DIST(NT-2,IA)/ZA
    Z4=Z2*TIM(NT-2)/ZA
    IF (IA.LE.5) Z4=Z2*TIMD(NT-2)
    DO 17 IB=1,NMO
      IF (NMOM(IB).LE.0) GO TO 17
      Z5=0.
      IG=NMOM(IB)+1
      DO 16 IC=1,IG
        IH=IG-IC
        ZE=IH+1
      Z5=Z5+(Z4**(IH))/GAMMA(ZF)
      Z6=IG
      ZM=ZA
    IF (IA.LE.5) ZW=1./ZC
    CORMOM(IB,IA)=(CORMOM(IB,IA)+IDIST(NT-2,IA)*ZW*GAMMA(Z6)*Z5)/(Z2**
      1IG))/Z3
  17 CONTINUE
  18 CONTINUE
  C***
  AMOM IS REDEFINED TO BE THE TRUNCATION CORRECTION
  DO 19 IA=1,6
  DO 19 IB=1,NMO
    AMOM(IB,IA)=CORMOM(IB,IA)-AMOM(IB,IA)
  RE-ESTABLISH PROPER NORMALIZATION
  ZC=1./ZC
  DO 20 IA=1,NT
    TIMD(IA)=TIMD(IA)*ZC
  DO 21 IA=1,NMO
    Z1=((ZA*1.E06)**(NMOM(IA)-1))*1.E06
    Z3=((ZC*1.E06)**(NMOM(IA)-1))*1.E06
    AMOM(IA,6)=AMOM(IA,6)*Z1
    CORMOM(IA,6)=CORMOM(IA,6)*Z1
  DO 21 IB=1,5
    AMOM(IA,IB)=AMOM(IA,IB)*Z3
  21 CORMOM(IA,IB)=CORMOM(IA,IB)*Z3
    ZA=1./ZA
    ZC=1./ZC
  DO 22 IA=1,NT
    DENS(IA)=DENS(IA)*ZA
  DO 22 IB=1,5
    DIST(IA,IB)=DIST(IA,IB)*ZC
  C***
  ADD EXTRAPOLATION CORRECTION TO ESTIMATE OF MULTIPLICATION CONSTAN
  IF (L(10).EQ.1) EFFK=ESTK*(1.+(FISS(NT)/(ALOG(FISS(NT-4)/FISS(NT))
    1))*(TIM(NT)-TIM(NT-4)))
  C***
  FLUX MEAN ENERGY
  CCSENER=0.
  CSENER=0.
  DO 23 IA=2,NM
    CCSENER=CCSENER+FLUX(IA)*EII(IA)*DU(IA)
    CSENER=CSENER+SFLUX(IA)*EII(IA)*DU(IA)
  23 RETURN
  INC

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**** NORMAL ****

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SUBROUTINE NORMAL
COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SELUX(200),RR(200),C(
171),SIGT(71),SIGF(71,5),SIGN(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
2,CB2(71),SIGTR(71),FISS(400),LPAK(400),CAPT(400),INFL(400),SPANS(4
300),DENS(400),FLCOR(200),TIM(400),U(201),DU(201),V(201),CV(201),FC
4201),DE(201),EII(201),VII(201),ANUNDR(71),NG(201),FVAR1(204),FVAR2
5(204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),AMDM(21,6),ATOM(10),BGF(26),PGSF(2
16),COMMONM(21,6),DEABN(26),DT(21),EABN(26),EMIN(5),ERR(21,2),EXM(21
2,2),H(12,12,5),L(30),NMCM(21),NNG(26),NORM(6),RHON(5),R(6),RHO(5),
3RI(4),RSS(26,6,4,5),SARG(5),SIGTB(26,5),SIGFB(26,5),SIGNB(26,5),SI
4GCR(26,5),SIGFB(26,5),AMUR(26,5),DBPR(26),SIGTRB(26),SIGTRT(26),SI
5GEP(5),SSSTAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),RSC(6),RSPR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CENTER,CONVC,CRIT,CSFNER,DDT,DTI,EFFK,ESTK,I,I
1A,IB,IC,ID,IE,IG,IH,IND,IP,PAK,J,JJ,JRCW,KROW,LL,MAXP,MAXI,MAXV,N,N
2A,NQ,NGR,NF,NINIT,NM,NMO,NP1,NP2,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
3OP,NVIR,NVM,OPCRIT,SCRIT,SQCRIT,SUMPOP,TIMWID,ZA,ZB,ZC,ZD,ZE,ZF,ZG
REAL LEAK,NORM
DIMENSION DTIM(400),TIMD(400),DINT(200,6),TINT(200),DIST(400,6)
1)
EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
1, (FISS,DIST)
NORMAL NORMALIZES TIME DEPENDENT DISTRIBUTIONS AND FLUXES
DC 1 IA=1,LL
DENS(IA)=0.
1 SDENS(IA)=0.
DC 3 IA=1,5
NORM(IA)=0.
DC 2 IB=1,NTM
2 NORM(IA)=NCPM(IA)+DIST(IB,IA)
IF (NORM(IA).GT.0.) NORM(IA)=1./NORM(IA)
3 CONTINUE
DC 4 IR=1,NTM
DD 4 IA=1,5
TDI=1./((TIM(IB+1)-TIM(IB))
4 DIST(IB,IA)=DIST(IB,IA)*NORM(IA)*TDI
DC 5 IA=1,5
5 DIST(NT,IA)=DIST(NT,IA)*NORM(IA)*TDI
IF (L(10).EQ.1) {STK=1./NORM(1)
NORM(6)=0.
DC 6 IA=2,NTM
6 NORM(6)=NORM(6)+DENS(IA)*(TIM(IA+1)-TIM(IA-1))
IF (NORM(6).GT.0.) NORM(6)=2./NORM(6)
DC 7 IA=1,NT
7 DENS(IA)=DENS(IA)*NORM(6)
FLUX IS NORMALIZED IN LETHARGY.
NORM(1)=0.
NORM(2)=C.
DC 8 IA=2,NVM
SFLUX(IA)=SFLUX(IA)*VII(IA)
FLUX(IA)=FLUX(IA)*VII(IA)
NORM(1)=NORM(1)+SFLUX(IA)
NORM(2)=NORM(2)+FLUX(IA)
SFLUX(IA)=SFLUX(IA)/DU(IA)
FLUX(IA)=FLUX(IA)/DU(IA)
8 IF (BUCKLE.GT.0.) NORM(1)=1./NORM(1)
NORM(2)=1./NORM(2)
DC 9 IA=1,NVM
SFLUX(IA)=SFLUX(IA)*NORM(1)
FLUX(IA)=FLUX(IA)*NORM(2)
9 RETURN
END

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5 SUBROUTINE OUTPUT
10 COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SFLUX(200),RR(200),C(
15 171),SIG(71,5),SIGN(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
20 2,DB2(71),SIGR(71),FISS(400),LFAK(400),CAP(400),TTL(400),SCNS(4
25 300),DENS(400),FLCOR(200),TIM(400),U(201),DU(201),V(201),FV(201),F
30 4201),R(201),FII(201),VII(201),ANUNOR(71),NG(201),FVAR1(204),FVAR2
35 5(204),FVAR3(204),FVAR4(204)
40 COMMON ALP(5),ALPHA(5),AMASS(5),AMOM(21,6),ATCM(10),RGF(26),PGSF(2
45 16),CORMOM(21,6),DEABN(26),DT(21),FABN(26),EMIN(5),FRR(21,2),EXM(21
50 2,2),H(12,12,5),L(30),NMOM(21),NNG(26),NORM(6),RHON(5),R(6),RHO(5),
55 3R(4),FSS(26,6,4,5),SARG(5),SIGTB(26,5),SIGFR(26,5),SIGNR(26,5),S
60 4GCAT(26,5),SIGBR(26,5),AMUB(26,5),DB2R(26),SIGTRH(26),SIGTR(26),S
65 50-P(5),SSSTAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),PSD(6),RSDR
70 6(2),RSDX(2)
75 COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DDT,DTI,FFFK,ESTK,I,I
80 1A,IR,IC,IO,IE,IG,IH,IND,IPEAK,J,JJ,JRCW,KROW,LL,MAXR,MAXT,MAXV,N,N
85 2A,ND,NGR,NF,NI,NII,NM,NMC,NPI,NP2,NP3,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
90 3OP,NVIR,NVM,OPCRIT,SCRIT,SOCPIT,SUMPOP,TIMWID,ZA,ZB,ZC,ZD,ZE
95 REAL LEAK,NORM
100 DIMENSION DTIM(400),TIMD(400),DINT(200,6),TINT(200),DIST(400,6)
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C***
SUMMARIZES RESULTS
WRITE(6,47)
WRITE(6,48) (TITLE(I),I=1,NTITLE)
IF (L(10).EQ.1) WRITE(6,49) EFKK
IF (L(10).EQ.1) WRITE(6,52) ESTK
WRITE(6,50) CCENER,CSENER
Z=TIM(NT)
ZA=SUMPPOP
ZB=SUMPPOP
ZC=NTOP+1
DO 1 IA=IB,NVM
1 SUMPPOP=SUMPPOP+PTP(IA)
ZB=1.-PTP(NVIR)-PTP(NVIR+1)-PTP(NVIR+2)-PTP(NVIR+3)-SUMPPOP
WRITE(6,51) Z,PTP(NVIR+1),PTP(NVIR+2),PTP(NVIR+3),PTP(NVIR),E(NVI
1R-1),SUMPPOP,ZB,ZA
IB=NVIR+3
WRITE(6,53)
WRITE(6,54) (PTP(IA),IA=1,IB)
WRITE(6,55)
WRITE(6,55) (FLUX(IA),IA=1,NVM)
WRITE(6,56)
WRITE(6,56) (SFLUX(IA),IA=1,NVM)
WRITE(6,23)
IF (L(16).EQ.1.OR.L(17).EQ.0) GO TO 3
3 CALCULATION OF RELATIVE STANDARD DEVIATION
DO 2 IA=2,6
2 RSD(IA)=SORT(ABS(CORMOM(2,IA))/(CORMOM(1,IA)**2)-1.)
RSDX(IA-4)=SORT(ABS(EXM(2,IA-4))/(EXM(1,IA-4)**2)-1.)
RSDR(IA-4)=RSD(IA)/RSDX(IA-4)-1.
3 IF (L(15).EQ.1) GO TO 5
DO 4 IA=1,4
4 IF (CORMOM(1,IA).GT.0.) RSD(IA)=SORT(ABS(CORMOM(2,IA)/CORMOM(1,IA)
1**2)-1.)
4 CCATINUF
4 CCATINUF
5 WRITE(6,24) FII(NVIR-1),E(NVIR-1)
WRITE(6,25) (NMOM(IA),CORMOM(IA,6),EXM(IA,2),AMOM(IA,6),ERR(IA,2)
1,NMOM(IA),CORMOM(IA,5),EXM(IA,1),AMOM(IA,5),FRR(IA,1),IA=1,NMO
WRITE(6,26) RSD(6),RSDX(2),RSDR(2),RSD(5),RSDX(1),RSDR(1)
WRITE(6,27)
OUTPUT-SPECIAL ROUTINE FOR COMPUTING RATIOS OF CALCULATED MOMENTS
IF (L(7).EQ.0) GO TO 7

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6      DC 6, IA=1,NMC
      CORMOM(IA,6)=CORMOM(IA,6)/E XM(IA,2)
      WRITE (6,28)
      WRITE (6,29) (NMC(IA),CORMOM(IA,6),IA=1,NMC)
      IF (L(15).EQ.1) GO TO 8

7      CONTINUE
      WRITE (6,23)
      WRITE (6,30)
      WRITE (6,31) (NMC(IA),CORMOM(IA,IB),IB=1,4),(AMCM(IA,IP),IP=1,4)
      IA=1,NMC)
      WRITE (6,32) (RSD(IA),IA=1,4)

8      CONTINUE
      IF (L(16).EQ.1) GO TO 10
      INTERPOLATED DISTRIBUTIONS
      IA=NIT
      IR=IA/50+1
      DC 9 IC=1,IR
      ID=50*(IC-1)+1
      IF=50*IC
      IF (IC.EQ.1R) IE=IA
      WRITE (6,33) DTI
      WRITE (6,34) (IG,TINI(IG),DINT(IG,6),DINT(IG,5),IG,IG=ID,IE)

9      CONTINUE
10     CONTINUE
      IF (L(15).EQ.1) GO TO 12
      INTERPOLATED DISTRIBUTIONS TO FISSION, LEAKAGE, CAPTURE, AND DEATH
      IA=NIT/50+1
      DC 11 IB=1,IA
      IC=50*(IB-1)+1
      ID=50*IB
      IF (IB.EQ.1A) ID=NIT
      WRITE (6,35) DTI
      WRITE (6,36) (IE,TINT(IE),(DINT(IE,IG),IG=1,4),IE,IE=IC,ID)

11     CONTINUE
12     CONTINUE

C***
13     GROUP AVERAGED FLUXES
      ZA=SQRT(3.204/1.674663E-12)
      NORM(1)=0.
      NORM(2)=0.
      IC=NG(1)
      IG=NG(NVIR-1)
      IF (IG.EQ.0) IG=NG(NVIR-2)
      IH=NG(3)
      IF (IH.EQ.0) IH=NG(4)
      IF (IH.EQ.1) IH=2
      IF (NG(2).NE.1) NNG(IH-1)=0
      IF (NG(2).EQ.0) NNG(IH-1)=1
      CC 14, IA=IH,IG
      RGF(IA)=0.
      RGSF(IA)=0.
      IC=NG(IA-1)+2
      IF=NG(IA)
      DO 13, IB=IC, ID
      RGF(IA)=RGF(IA)+FLUX(IB)*DU(IB)
      RGSF(IA)=RGSF(IA)+SFLUX(IB)*DU(IB)

13     CONTINUE
      ZR=ALOG(CABN(IA-1)/CABN(IA))
      BGF(IA)=RGF(IA)+FLUX(IC-1)*ALOG(CABN(IA-1)/CABN(IA))+FLUX(IC+1)*AL)
      RGSF(IA)=RGSF(IA)+SFLUX(IC-1)+SFLUX(IC+1)
      1*ALOG(IE/ID)/CABN(IA))
      NORM(1)=NORM(1)+RGF(IA)
      NORM(2)=NORM(2)+RGSF(IA)
      BGF(IA)=RGF(IA)/ZR
      RGSF(IA)=RGSF(IA)/7B

14     CONTINUE
      RGF(1)=0.
      RGSF(1)=0.
      IF (NG(2).NE.1) GO TO 16

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00 15 IB=2, ID
    BGF(1)=BGF(1)+FLUX(IB)*DU(IB)
    BGSF(1)=BGSF(1)+SFLUX(IB)*DU(IB)
15 CONTINUE
    ZB=ALOG(10.506/EARN(1))
    BGF(1)=BGF(1)+FLUX(ID+1)*ALJG(F(ID)/EARN(1))
    BGSF(1)=BGSF(1)+SFLUX(ID+1)*ALDG(F(ID)/EARN(1))
16 CONTINUE
    NORM(1)=1./(NORM(1)+BGF(1))
    NORM(2)=NORM(2)+BGSF(1)
    IF (NORM(2).NE.0.) NORM(2)=1./NORM(2)
    BGF(1)=BGF(1)/ZB
    BGSF(1)=BGSF(1)/ZB
    DO 17 IA=1H, IG
    BGSF(IA)=BGSF(IA)*NORM(2)
    BGF(IA)=BGF(IA)*NORM(1)
17 CONTINUE
    WRITE (6,37)
    IF (IH.EQ.2) WRITE (6,38) EARN(1), BGF(1), BGSF(1)
    WRITE (6,39) (IA, EARN(IA-1), EARN(IA), BGF(IA), BGSF(IA), IA=1H, IG)
    C***
    DETAILED TIME DISTRIBUTIONS ARE WRITTEN OPTIONALLY
    IF (L(18).EQ.0) GO TO 21
    IF (L(18).EQ.1) GO TO 19
    IA=NT
    IB=IA/50+1
    DO 18 IC=1, IB
    ID=50*(IC-1)+1
    IC=50*IC
    IF (IC.EQ.9) GO TO 18
    IF (IC.EQ.18) IE=IA
    WRITE (6,40)
    WRITE (6,41) (IG, TIM(IG), DENS(IG), SDENS(IG), IG, IG=ID, IF)
18 CONTINUE
    IF (L(15).EQ.1) GO TO 21
19 CONTINUE
    DO 20 IC=1, IB
    ID=50*(IC-1)+1
    IC=50*IC
    IF (IC.EQ.9) GO TO 20
    IF (IC.EQ.18) IE=IA
    WRITE (6,42)
    WRITE (6,43) (IG, TIMD(IG), (DIST(IG, IH), IH=1, 4), IG, IG=ID, IE)
20 CONTINUE
21 CONTINUE
    IF (L(14).EQ.0) GO TO 22
    WRITE (7,43) (TITL(1), I=1, NTITL), (ATOM(2*I-1), ATOM(2*I), I=1, N(1)
    VSIG=V(NVIR-1)*SIG*8(25,1)
    IA=NVIR+3
    ID=5
    IE=6
    IF (L(17).EQ.1) ID=2
    IF (L(16).EQ.1) ID=1
    IF (L(16).EQ.1) IE=4
    WRITE (7,44) ID, IE, NIT, NP3, IA
    WRITE (7,45) DT, BUCKLE, GEFK, VSIG
    PUNCH INTERPOLATED DISTRIBUTIONS
    WRITE (7,46) (TINT(IB), IB=1, NIT)
    WRITE (7,46) ((DINT(IB, IC), IB=1, NIT), IC=ID, IF)
22 CONTINUE
    RETURN
    C***
    C***
23 FORMAT (11, 35X, 'TABLE OF MOMENTS OF DISTRIBUTIONS', /, 41X, '(UNITS
    1ARE MICROSECONDS)')
24 FORMAT ('0', //, 10X, 'NEUTRON DENSITY AT ', 1PE10.3, ' EV.', 12X, '5LCEW1
    1NG DOWN DENSITY AT ', 1PE10.3, ' EV.', 12X, 'N CALC. ASym. F.C.
    2YM.FERROR', //)
25 FORMAT ('0', 1, 'RSD', 1P2E13.4, 1P11.2, 2PF8.3))
26 FORMAT ('0', 1, 'RSD', 1P2E13.4, 2PF19.3, 1P17.4, 1PF13.4, 2PF19.3)

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SURROUT INE PARAM
COMMON P(74,74),POP(74,2),PIP(204),FLUX(200),SELUX(200),PF(200),G(
171),SIGT(71),SIGF(71,5),SIGN(71,5),SIGC(71,5),SIGR(71,5),AMU(71,5)
2,CB2(71),SIGTR(71),FISS(400),LEAK(400),CAPT(400),TOTL(400),SDEAS(4
300),DENS(400),FLCOR(200),TIM(400),U(201),V(201),FV(201),E(
4201),DE(201),Z(201),V1(201),ANUNOR(71),NG(201),FVAR1(204),FVAR2
5(204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),AMQM(21,6),ATOM(10),RGF(26),BGSE(2
16),CORMOM(21,6),DEABN(26),DT(21),EARN(26),EMIN(5),ERR(21,2),EXM(21
2,2),H(12,12,5),L(30),NMOM(21),NNG(26),NORM(6),RHO(5),R(6),RHO(5),S
3RI(4),RSS(26,6,4,5),SARG(5),SIGTR(26,5),SIGF8(26,5),SIGNR(26,5),S
4GCR(26,5),SIGC8(26,5),AMUB(26,5),DB28(26),SIGTR(26),SIGTR(26),SI
5GEP(5),SSTAR(5),SZFR(5),SCALE(26,5),TITLE(72),ALAM(5),RSD(6),RSCR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DDT,DII,EFFK,ESTK,I,I
1A,IH,IC,ID,IE,IG,IH,IND,IPEAK,J,JJ,JRCW,KROW,LL,MAXR,MAXT,MAXV,N,N
2A,ND,NGR,NF,NI,NIT,NM,NPC,NP1,NP2,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
3OP,NVIR,NVM,OPCRIT,SCRIT,SCCRIT,SUMPOP,TIMWID,ZA,ZR,ZC,ZD,ZE
REAL LEAK,NORM
DIMENSION DTIM(400),TIMD(400),JINT(200,6),TINT(200),DIST(400,6
1)
EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
1,(FISS,DIST)
C***
C***
C***
C***
THIS SUBROUTINE DEFINES ALL PARAMETERS OF THE STATE STRUCTURE AND
ARRANGEMENT NOT PREVIOUSLY DEFINED.
C***
ZA=SQRT(3.204/1.674663E-12)
V(1)=ZA*SQRT(E(1))
TEST TO SEE WHICH OF U AND DU HAVE BEEN DEFINED
IF (DU(2).EQ.0.) GO TO 2
U(1)=0.
C***
C***
C***
C***
THE NEXT EIGHT STATEMENTS DEFINE STATE BOUNDARIES WHEN THE
LETHARGY INTERVALS DU ARE DEFINED IN INPUT DATA (TO LOG BASE 10).
LETHARGY IS CONVERTED TO BASE F.
DO 1 I=2,NVIR
DU(1)=2.3025851*DU(I)
U(1)=U(I-1)+DU(I)
E(1)=E(I-1)*EXP(-U(I))
V(1)=ZA*SQRT(E(1))
V(1)=ZA*SQRT(E(1))
DV(1)=V(I-1)-V(I)
DE(1)=E(I-1)-E(I)
GO TO 4
1 GO TO 4
C***
C***
C***
C***
THE NEXT SEVEN STATEMENTS DEFINE STATE BOUNDARIES WHEN THE
LETHARGIES U HAVE BEEN DEFINED.
DO 3 I=2,NVIR
U(1)=U(I)*2.3025851
E(1)=E(I)*EXP(-U(I))
V(1)=ZA*SQRT(E(1))
DU(1)=U(I)-U(I-1)
DV(1)=V(I)-V(I-1)
DE(1)=E(I)-E(I-1)
CONTINUE
3 CONTINUE
C***
C***
C***
C***
IN THE ROUTING BELOW THE LOCATION OF STATES WITHIN BROAD GROUPS IS
SPECIFIED BY NG. IF NG(I)=0 STATE I SPANS THE BOUNDARY OF A BROAD
GROUP. NNG(I) SPECIFIES THE INDEX OF THE LOWEST ENERGY STATE THAT
LIES COMPLETELY IN I.
ID=1
NNG(1)=0
DO 6 IA=2,NVIR
DO 5 IB=ID,NGR
IF (EABN(IB).GT.F(IA-1)) GO TO 5
NG(IA)=IB
IF (EABN(IB).GT.E(IA)) NG(IA)=0
NNG(IB)=IA-1
IF=IB
GO TO 6
6

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0 NSCAT=1
1 NMG=2
2 NMOM(1)=1
3 NMCN(2)=2
4 CRIT=.001
5 UPCRIT=.4
6 SQRIT=.2
7 SCRT=.001
8 CONVC=.000C01
9 TIMID=0.
10 YIM(1)=0.
11 DT(IA)=0.
12 DT(IA)=0.
13 BUCKLF=0.
14 F(1)=1.05EC7
15 CL(1)=0.
16 DV(1)=0.
17 DU(1)=0.
18 *** VALUES OF MICROSCOPIC CROSS SECTIONS AND SCALE FACTORS.
19 DEFAULT
20 DO 3 IA=1,ND
21   SIGTR(IB,IA)=0.
22   SIGFR(IB,IA)=0.
23   SIGNR(IB,IA)=0.
24   SIGCR(IR,IA)=0.
25   SIGFB(IR,IA)=0.
26 3 AMUR(IR,IA)=0.
27 DO 4 IA=1,NGR
28   BGSF(IA)=0.
29 DO 4 IB=1,ND
30   SCALE(IA,IB)=1.
31 *** DEFAULT VALUE OF RESONANCE SELF-SHIELDING COEFFICIENTS.
32 DO 5 IA=1,ND
33   IC=1.4
34 DO 5 IC=1.6
35 DO 5 ID=1,NGR
36 RSS(ID,IC,IB,IA)=1.
37 LASTNO=0
38 REWIND 1
39 DO 6 IA=1,ND
40   NOLIST(IA)=0
41 READ (5,37)END=30) (TITLE(IA),IA=1,NTITLE)
42 READ (5,35) (L(IA),IA=1,30)
43 L(2) IS USED TO CHECK FOR CONSISTENCY IN THE DATA SET
44 IF (L(2).NE.1) WRITE (6,36)
45 IF (L(2).NE.1) GO TO 30
46 READ (5,32) (NOLIST(IA),IA=1,ND)
47 NAMELIST NUMBERS MAY BE USED TO OVERRIDE CROSS SECTIONS READ FROM
48 EXTERNAL FILES. IF NOLIST(1)=0 THE EXTERNAL FILES ARE NOT READ.
49 IF (NOLIST(1).EQ.0) GO TO 16
50 DO 7 IA=1,ND
51 IF (NOLIST(IA).GT.NUMAXI) GO TO 15
52 IF (NOLIST(IA).EQ.0) GO TO 8
53 7 CONTINUE
54 GO TO 9
55 8 NI=IA-1
56 9 CONTINUE
57 ISOTOPE NUMBERS ARE SEQUENCED NUMERICALLY EXCEPT THAT HYDROGEN
58 (ISOTOPE 1) MUST APPEAR AT THE END OF THE LIST.
59 DO 10 IA=1,NI
60   NNG(IA)=NOLIST(IA)
61 *** TEMPORARILY PRESERVE INPUT ORDERING OF ISOTOPE LIST.
62 IF (NI.EQ.1) GO TO 13
63 IC=NI-1
64 DO 11 IA=1,IC
65   IC=NI-IA
66 DO 11 IB=1,ID
67   IF (NOLIST(IB+1).GT.NOLIST(IB)) GO TO 11

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635 IE=NOLIST(IB)
640 NOLIST(IR)=NOLIST(IB+1)
645 NOLIST(IB+1)=IE
650 CONTINUE
655 IF (NOLIST(1).NE.1) GO TO 13
660 PUT HYDROGEN AT END OF LIST
665 DO 12 IA=1,IC
670 NOLIST(IA)=NOLIST(IA+1)
675 NOLIST(NI)=1
680 CONTINUE
685 DO 14 IA=1,NI
690 NC=IA
695 CALL GETIT (NOLIST(IA),NC,LASTNO)
700 GO TO 16
705 WRITE (6,33)
710 GO TO 30
715 CONTINUE
720 IN NAMELIST FILE 'NUMBERS' POSITIVE MOMENTS MUST BE LISTED FIRST
725 FOR VARIABLE 'NMOM' BEGINNING WITH NMOM=1,2...
730 READ (5,NUMBERS)
735 DENSITIES AND SCALE FACTORS ARE RE-ORDERED TO CONFORM TO PROPER
740 ISOTOPE SEQUENCE
745 IF (NI.EQ.1) GO TO 20
750 IF=NI-1
755 DO 19 IA=1,IE
760 IF (NOLIST(IA).EQ.NNG(IA)) GO TO 19
765 ID=IA+1
770 DO 18 IB=ID,NI
775 IF (NOLIST(IA).NE.NNG(IB)) GO TO 18
780 ZA=RHO(IB)
785 RHO(IB)=RHO(IA)
790 RHO(IA)=ZA
795 IC=NNG(IB)
800 NNG(IB)=NNG(IA)
805 NNG(IA)=IC
810 DO 17 IC=1,NGR
815 ZA=SCALE(IC,IB)
820 SCALE(IC,IB)=SCALE(IC,IA)
825 SCALE(IC,IA)=ZA
830 GO TO 19
835 CONTINUE
840 CONTINUE
845 CONTINUE
850 PARAMETER AND VECTOR INITIALIZATIONS FOLLOW
855 RR(NVIR)=0.
860 CU(NVIR)=1.E06
865 LL=1
870 NM=N-1
875 NP1=N+1
880 NP2=N+2
885 NP3=N+3
890 DO 21 IA=1,N
895 NG(IA)=0
900 IPEAK=2
905 NTIME=1
910 NTOP=0
915 NT=0
920 NVM=NVIR-1
925 SUMPOP=0.
930 THE ENERGY STRUCTURE OF THE MULTIGROUP CROSS SECTIONS IS DEFINED.
935 THIS PROGRAM ASSUMES THE STRUCTURE OF THE RUSSIAN ABN SET.
940 EABN(1)=6.5E06
945 EABN(2)=4.0E06
950 EABN(3)=2.5E06
955 EABN(4)=1.4E06
960 EABN(5)=8.0E05
965 EABN(6)=4.0E05
970 EABN(7)=2.0E05
975 EABN(8)=1.0E05
980 EABN(9)=4.65E04

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REAL LEAK,NORM
DIMENSION DTIM(400), TIMD(400), DINT(200,6), TINT(200), DIST(400,6)
1) EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
1, (FISS,DIST)

SQUARES THE TRANSITION PROBABILITY MATRIX.
NEWLY SQUARED MATRIX ELEMENTS ARE INITIALLY STORED IN TRANSPOSED
ORDER IN LOWER TRIANGLE OF MATRIX.

CALL CANCEL (2)
IF (JROW.GE.NM) GO TO 7
SQUARING ROUTINE WHEN SOME ROWS ARE NOT YET DEFINED.
ITERATE ON ROWS
DO 6 IA=1, JROW
IC=IA+1
ITERATE CN COLUMNS
DO 4 IB=ID,NP3
IF (IB.GT.JROW) GO TO 2
DO 1 IC=IA,IR
1 P(IB,IA)=P(IB,IA)+P(IA,IC)*P(IC,IB)
2 GO TO 4
DO 3 IC=IA,JROW
P(IB,IA)=P(IB,IA)+P(IA,IC)*P(IC,IB)
3 CONTINUE
DO 5 IB=KROW,NP3
NEUTRONS WHICH HAVE ALREADY SCATTERED BEYOND JROW REMAIN IN THE
SAME STATE UNTIL ITS ROW IS DEFINED.
4 P(IB,IA)=P(IB,IA)+P(IA,IB)
5 P(IA,IA)=P(IA,IA)+P(IA,IA)
6 GO TO 10
SQUARING ROUTINE WHEN ALL ROWS ARE DEFINED.
DO 9 IA=1,NM
IC=IA+1
DO 8 IB=ID,NP3
DO 8 IC=IA,IR
P(IA,IA)=P(IA,IA)+P(IA,IA)*P(IA,IB)
9 P(IA,IA)=P(IA,IA)+P(IA,IA)*P(IA,IA)
TRANSPOSE ELEMENTS FOR ALL DEFINED ROWS.
DO 11 IA=1,JPOW
IC=IA+1
DO 11 IB=ID,NP3
P(IA,IB)=P(IB,IA)
11 RETURN
END

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**** STAWID ****

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SUBROUTINE STAWID (*)
COMMON B(74,74),POP(74,2),PTP(204),FLUX(200),SELUX(200),RR(200),G(
171),SIGT(71),SIGF(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
2,CB2(71),SIGTR(71),FISS(400),LEAK(400),CAPT(400),ICTL(400),SCENS(4
300),DENS(400),FLCOR(200),TIM(400),U(201),DV(201),F(
4201),DE(201),EII(201),VII(201),ANUNGR(71),NG(201),FVAR1(204),FVAP2
5(204),FVAP3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),ATOM(10),BGE(26),PGSF(2
16),CORMOM(21,6),DEARN(26),ZMIN(5),ERR(21,2),XXM(21
2,2),H(12,12,5),L(30),NMODM(21),NNGT(26),RHON(6),R(6),PHC(5),S
3RI(4),RSS(26,6,4,5),SAFE(5),SIGTA(26,5),SIGFR(26,5),SIGNP(26,5),SI
4GCB(26,5),SIGER(26,5),AMUR(26,5),DB2R(26),SIGTRR(26),SIGTRI(26),SI
5GEP(5),SSTAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),RSD(6),RSDR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DOI,DTI,EFFK,I,I
1A,IR,IC,IB,IE,IG,TH,IND,IPEAK,I,II,IPOW,KROW,L,MAXR,MAXT,MAXV,N,N

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445 P IF (L(4)-F9.0) GO TO 9
450 P THE NEXT FIVE STATEMENTS DEFINE THE STATE STRUCTURE FROM F(NVM)
455 C*** UPWARD IF L(4)=1.
460 C*** ID=NVIR-2
465 P DO 7 IA=3, ID
470 P DU(IA)=DU(2)
475 P U(NVM)=DU(2)
480 P U(NVM)=0.
485 P DO 8 IA=2, NVM
490 P U(NVM)=U(NVM)+DU(IA)
495 C*** F(1)=F(NVM)*10.**U(NVM)
500 P GO TO 12
505 P THE NEXT FIVE STATEMENTS DEFINE THE STATE STRUCTURE FROM E(1)
510 C*** DOWNWARD (DEFAULT OPTION).
515 P DO 10 IA=3, NVM
520 P DU(IA)=DU(2)
525 P U(NVM)=0.
530 P DO 11 IA=2, NVM
535 P U(NVM)=U(NVM)+DU(IA)
540 P U(NVM)=0.
545 P DO 12 CONTINUE
550 C*** RETURN 1
555 P RETURN 1
560 C***
565 C***
570 P 14 FORMAT ('1', ///, 20X, 'STATE WIDTH OPTIMIZATION DATA', ///, 10X, 'THE
575 P INITIAL LETHARGY DECREMENT (LOG10) IS', E12.5, ///, 10X, '11', 'STATE(S)
580 P 2PER SCATTERING INTERVAL', ///, 10X, 'THE CONVERGENCE CRITERIA IS', F9.5
585 P 3)
590 P 15 FORMAT ('0', ///, 'AT ITERATION NUMBER', I3, ' THE ERROR IS', F9.5, '
595 P 16 FORMAT ('0', ///, 'F8.5', ' AND THE NEW LETHARGY WIDTH IS', E12.5)
600 P 1 MINATED*****
605 P 1 ENC
610 P

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5 SUBROUTINE TRAN
10 COMMON P(74,74), POP(74,2), PTP(204), FLUX(200), SFLUX(200), RR(200), G(
15 171), SIGT(71), SIGF(71,5), SIGC(71,5), SIGE(71,5), AMU(71,5),
20 2, DB2(71), SIGTR(71), FISS(400), LFAR(400), CAPT(400), TOTL(400), SCENS(4
25 300), DENS(400), FLCOR(200), TIM(400), U(201), DU(201), V(201), DV(201), E(
30 4201), DE(201), VII(201), ANUNCR(71), NG(201), FVAR1(204), FVAR2
35 5(204), FVAR3(204), FVAR4(204)
40 COMMON ALP(5), ALPHA(5), AMASS(5), AMOM(21,6), ATOM(10), BGF(26), PGSF(2
45 16), CORMOM(21,6), DEABN(26), DT(21), EABN(26), EMIN(5), ERR(21,2), EXM(21
50 2,2), H(12,12,5), L(30), NMMOM(21), NNG(26), NORM(6), RHON(5), R(6), RHC(5),
55 3PI(4), RSS(26,6,4,5), SAPG(5), SIGTR(26,5), SIGFR(26,5), SIGNB(26,5), SI
60 4GCB(26,5), SIGER(26,5), AMUB(26,5), DB2B(26), SIGTRB(26), SIGTRT(26), SI
65 5GCP(5), SSTAR(5), SZER(15), SCALE(26,5), TITLE(72), ALAM(5), RSC(6), RSR
70 6(2), RSDX(2)
75 COMMON ACCUM, BUCKLE, CCENER, CONV, CRIT, CSFNER, DDT, DTI, EFKK, CSTK, I, I
80 1A, IB, IC, ID, IF, IG, IH, IND, IPEAK, J, JJ, JROW, KROW, LL, MAXR, MAXI, MAXV, N, N
85 2A, ND, NG, NF, NI, NJ, NM, NMC, NP2, NP3, NSCAT, NT, NTIME, NTITLE, AT, M, NT
90 3OP, NVIR, NVM, OPCRIT, SCRTT, SQCRIT, SUMP, TIMWID, ZA, ZR, ZC, ZC, 7
95 REAL LEAK, NORM
100 DIMENSION DTIM(400), TIMD(400), DINT(200,6), TINT(200), CIST(400,6
105 1)
110 EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(401),DINT), (P(3201),TINT)
115 1, (FISS,DIST)
120
125 TRAN COMPUTES THE TRANSFER COEFFICIENT MATRIX WITH THE ASSUMPTION
130 OF CONSTANT CROSS SECTIONS ACROSS EACH STATE. THE ARRAY CF TRANS
135 FOR COEFFICIENTS IS COMPUTED IN TRANSPOSED FORM SO IT MAY BE
140 STOPPED AT ANY TIME. LOWER TRIANGULAR OPTION DELETES THE REAL MATRIX

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C*** DOUBLE PRECISION ZAD,ZBC,ZCD,ZDD,ZED,ZFD
C*** DIMENSION UB(26), DUB(26)
C*** LETHARGIES OF BROAD GROUP BOUNDARIES AND LETHARGY WIDTHS OF BROAD
C*** GROUPS ARE DEFINED FOR USE IN LETHARGY WEIGHTING OF INELASTIC
C*** CROSS SECTIONS
UB(1)=ALOG(E(1)/EARN(1))
DUB(1)=UB(1)
DO 1 IA=2,NGR
UB(IA)=ALOG(E(1)/EARN(IA))
DUB(IA)=UB(IA)-UB(IA-1)
C*** NII= NUMBER OF NON-HYDROGENOUS ISOTOPES.
NII=N1
NZX=(N+NTOP)
E(N+NTOP)=0.
IF (AMASS(NI).EQ.1.) NII=N1-1
RR(1)=1.
C*** INITIALIZE THE MATRIX
DO 2 IA=1,NP3
DO 2 IR=1,NP3
P(IA,IR)=0.
C*** STATE 1 IS A DELTA FUNCTION AND IS POPULATED ONLY WHEN CURRENTLY
C*** BEING USED AS A SOURCE STATE (N1=1).
N1=2
IF (NTOP.EQ.0.AND.L(11).EQ.1) N1=1
DO 39 IA=N1,NM
IA IS THE ROW INDEX ON THE REAL MATRIX.
IAPI IS THE ROW INDEX ON THE VIRTUAL MATRIX.
IAP=IA+NTOP
NAG=NG(IAP)
IF (NAG.EQ.0) NAG=NG(IAP-1)
C=0.
DO 3 IC=1,N1
FIRST FACTOR ON RIGHT SIDE OF EQ. 36.
SIGEP(IC)=SIGE(IA,IC)*(SCALE(NAG,IC)**2)/(1.-ALPHA(IC))
C=C+SIGEP(IC)
C*** EQ. 41
ALP(IC)=(1.-ALPHA(IC))/SCALE(NAG,IC))
3 EMIN(IC)=ALP(IC)
IF (IA.EQ.1) GO TO 8
C*** AVERAGES FOR STATES 2 TO NM
ZAC=E(IAP-1)/E(IAP)
ZBC=OLOG(ZAD)
ZCD=1./(ZAD-1.)
IF (N11.EQ.0) GO TO 4
C*** EQ. 44
RR(IAP)=(ZAD**C-ZAD)/((ZAD-1.)*(C-1.))
EQ. 45
EII(IAP)=(E(IAP-1)*C*(ZAD**(C-1.-1.))/(C-1.)*(ZAD**C-1.))
GO TO 5
C*** EQS. 44 AND 45 MODIFIED FOR PURE HYDROGEN.
RR(IAP)=(ZAD*ZBD)/(ZAD-1.)
4 EII(IAP)=(E(IAP-1)*ZBD)/(ZAD-1.)
5 CONTINUE
VI=1.
C*** EXPCTATION OF INVERSE VELOCITY AS IN TABLE III
DO 7 IB=2,6
ZCD=0.
IC=IB-1
DO 6 IC=1,IB
ZCD=ZCD+((ZBD/2.)*E(IC)/DGAMMA(ZED)
6 ZCD=ZCD+((18-1.))*((1.-1.)/(ZAD**5-1.))*ZDD)
7 VI=VI+((2.*C)**(IB-1.))*((1.-1.)/(ZAD**5-1.))*ZDD)
VI=(VI*2.)/(RR(IAP)*V(IAP-1)+V(IAP))
GO TO 9
C*** EII(1+NTOP)=E(1+NTOP)
VI=1./V(1+NTOP)
VII(IAP) IS EFFECTIVELY THE MEAN VELOCITY OF NEUTRONS IN STATE IAP
9 VII(IAP)=1./VI

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505 EQ. B1.
510 G(IA)=SIGT(IA)*VII(IAP)
515 IF (IA.EQ.1) GO TO 14
520 IF (NI.EQ.0) GO TO 11
525 DO 10 IB=1,NI
530 EQ. A12. ALAM = N SUR S,K.
535 ALAM(IB) = ((C-1.)*(ZAD**C-1.))/(C*(ZAD**C-ZAD))-ALP(IB)/(1.-ALP(I
540 IB))
545 EQ. A10
550 10 FMIN(IB)=E(IAP)*((1.-C*(ALP(IB)*ZAD)**2)*(ZAD**C-2.)-1.))/((C-2
555 2**C-1.))-1.)
560
565 11 CONTINUE
570 IF THE LAST ISOTOPE (NI) IS HYDROGEN--
575 IF (NI.NE.NII) ALAM(NI)=E(IAP)/EII(IAP)
580 IF (NI.NE.NII) EMIN(NI)=0.
585 ID=IA+1
590 DO 13 IC=1,NI
595 FIRST FACTOR IN EQ. A11
600 HS=(SCALE(NAG,IC)*ALAM(IC))/(E(IAP)-EMIN(IC))
605 DO 12 IB=IC,N
610 IBP=IB+NTOP
615 EQ. A11D AND SECOND TERM OF EQ. 37
620 IF (FMIN(IC).LT.E(IRP)) P(IB,IA)=P(IB,IA)+SIGE(IA,IC)*S*DE(IBP)
625 EQ. A11C
630 IF (EMIN(IC).GT.E(IRP-1)) GO TO 13
635 EQ. A11B AND SECOND TERM OF EQ. 37
640 IF (FMIN(IC).GE.E(IRP)) P(IB,IA)=P(IB,IA)+SIGE(IA,IC)*S*(E(IRP-1)
645 1-EMIN(IC))
650 11 CONTINUE
655 12 CONTINUE
660 GO TO 18
665 13 CONTINUE
670 SPECIAL ROUTINE FOR ROW 1.
675 DO 15 IC=1,NI
680 15 EMIN(IC)=EMIN(IC)*EII(NTOP+1)
685 ID=IA+1
690 DO 17 IC=1,NI
695 DO 16 IB=ID,N
700 IBP=IB+NTOP
705 EQS. 39
710 IF (EMIN(IC).LT.E(IRP)) P(IB,1)=P(IB,1)+(SIGEP(IC)*DE(IBP))/EII(NT
715 OP+1)
720 IF (EMIN(IC).GT.E(IRP-1)) GO TO 17
725 IF (EMIN(IC).GE.E(IRP)) P(IB,1)=P(IB,1)+(SIGEP(IC)*(E(IRP-1)-EMIN(
730 IC))/EII(NTOP+1))
735 16 CONTINUE
740 17 CONTINUE
745 18 CONTINUE
750 IF (L(12).EQ.0) GO TO 36
755 THE FOLLOWING ROUTINE, TERMINATING AT STATEMENT NUMBER 35,
760 COMPUTES INELASTIC SCATTERING CONTRIBUTIONS TO THE TRANSFER
765 COEFFICIENTS. BUT THE COMPLEXITY OF THE ROUTINE ARISES FROM THE
770 NEED TO DISTRIBUTE BROAD GROUP INELASTIC TRANSFER PROBABILITIES
775 OVER MANY STATES WHOSE BOUNDARIES WILL GENERALLY NOT BE ALIGNED
780 (THAT IS, COINCIDENT) WITH BROAD GROUP BOUNDARIES. THE WISE
785 USER WILL TAKE THE CORRECTNESS OF THE FOLLOWING ON FAITH, AND
790 NOT ATTEMPT TO VERIFY IT.
795
800 IF THE ROW IAP LIES BELOW THE INELASTIC THRESHOLD BYPASS WHAT
805 FOLLOWS. INELASTIC SCATTERING FROM ROW 1 OF THE VIRTUAL MATRIX
810 IS NOT ALLOWED. THUS IF A DELTA FUNCTION SOURCE IS DESIRED WHEN
815 INELASTIC SCATTERING IS PRESENT IT SHOULD BE DEFINED IN STATE 2
820 BY READING THE VECTOR PTD(2)=1. IN NAMELIST FILE 'NUMBRS'.
825 IF (IAP.GT.NNG(11).OR. IA.EQ.1) GO TO 36
830 IAJ=NG(IAP)
835 IAJ=IAJ+1
840 IC=IA+1
845 ROW 12 AND COLUMN 12 OF HARRAY MUST CONTAIN ZEROES.
850

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00 19 IB=1,12
19 H(IB,12,IC)=0.
00 20 IB=1,12.
20 H(12,IB,IC)=0.
21 CONTINUE
C*** STATEMENTS THROUGH 28 ARE EXECUTED IF STATE IAP STRADDLES A
C*** BRCD GROUP BOUNDARY.
IF (IAJ.NE.0) GO TO 29
IAJ=NG(IAP-1)
IAJ1=IAJ+1
22 P(IA,IA)=P(IA,IA)+SIGNB(IAJ,IC)*W1*(H(IAJ,IAJ,IC)+H(IAJ,IAJ+1,IC))*
1((U(IAP)-UB(IAJ))/DUB(IAJ+1))/DE(IAJ))/SIGNB(IAJ+1,IC)*H(IAJ+1,IAJ+1,IC)*
2(EABN(IAJ)-UB(IAJ))/DE(IAJ))/SIGNB(IAJ+1,IC)*H(IAJ+1,IAJ+1,IC)*
C*** TEMPORARY RENORMALIZATION. SEE STATEMENT ---(111).
P(IA,IA)=P(IA,IA)/(1.-E(IAJ)/FII(IAJ))
IF=NG(NG(IAJ+1))-NTOP
IE IS THE REAL INDEX OF THE LOWEST STATE IN GROUP IAJ NOT CN
C*** THE GROUP BOUNDARY.
DO 23 IB=10, IF
IBP=IB+NTOP
ZF=W1*DU(1BP)/DUB(IAJ1)
DO 23 IC=1, NI
23 P(1B,IA)=P(1B,IA)+ZE*SIGNB(IAJ,IC)*H(IAJ,IAJ1,IC)
C*** INELASTIC TRANSFER TO THE STATE LYING ON THE LOWER BOUNDARY OF IAJ
DO 24 IC=1, NI
24 P(IE+1,IA)=P(IE+1,IA)-W2*SIGNB(IAJ1,IC)*H(IAJ1,IAJ+2,IC)*(UB(IAJ1)
1-(U(IE+1+NTOP))/DUB(IAJ+2))-W1*SIGNB(IAJ,IC)*H(IAJ,IAJ1,IC)*(U(IE+N
2TOP)-UB(IAJ1))/DUB(IAJ1)+H(IAJ,IAJ+2,IC)*(UB(IAJ1)-U(IE+1+NTOP))/E
3UB(IAJ+2))
C*** INELASTIC TRANSFER OUT OF GROUP IAJ TO STATES NOT LYING ON GROUP
C*** BOUNDARIES.
IF (IAP.GE.NNG(10)) GO TO 36
IF=IE+2
IG=NG(11)-NTOP+1
DO 28 IR=IE, IG
IBP=IB+NTOP
IBJ=NG(1BP)
IF (1BJ.EQ.0) GO TO 26
ZF=CU(1BP)/DUB(1BJ)
DO 25 IC=1, NI
25 P(1B,IA)=P(1B,IA)+ZE*(W1*SIGNB(IAJ,IC)*H(IAJ,1BJ,IC)+W2*SIGNB(IAJ1
1,IC)*H(IAJ1,1BJ,IC))
GO TO 28
26 IBJ=NG(1BP+1)
C*** INELASTIC TRANSFER OUT OF GROUP IAJ TO STATES ON GROUP BOUNDARIES
W3=-((U(1BP-1))-UB(1BJ-1))/DUB(1BJ-1)
W4=-((UB(1BJ-1))-U(1BP))/DUB(1BJ)
DO 27 IC=1, NI
27 P(1B,IA)=P(1B,IA)+W1*SIGNB(IAJ,IC)*(W3*H(IAJ,1BJ-1,IC)+W4*H(IAJ,1BJ,IC))+
1J,IC))+W2*SIGNB(IAJ1,IC)*(W3*H(IAJ1,1BJ-1,IC)+W4*H(IAJ1,1BJ,IC))
28 CONTINUE
GO TO 36
C*** STATEMENTS THROUGH 36 ARE EXECUTED IF STATE IAP DOES NOT LIE
C*** ON A GROUP BOUNDARY.
CONTINUE
DO 30 IC=1, NI
30 IF FINAL STATE LIES IN SAME GROUP AS STATE IAP DOWNSCATTER IS
P(IA,IA)=P(IA,IA)+SIGN(IA,IC)*H(IAJ,IAJ,IC)
P(IA,IA)=P(IA,IA)/(1.-F(IAJ)/FII(IAJ))
IF=NG(NG(IAJ))-NTOP+1
ZF=-((UB(IAJ))-U(IE+NTOP))/DUB(IAJ1)
DO 31 IC=1, NI
31 FINAL STATE(IE) LIES ON BOUNDARY OF GROUP NG(IAJ).
P(1E,IA)=P(1E,IA)+SIGN(IA,IC)*H(IAJ,IAJ1,IC)*ZE
31 P(1E,IA)=P(1E,IA)+SIGN(IA,IC)*H(IAJ,IAJ1,IC)*ZE

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C***
C*** INPUT AND COMPUTED DATA FOR PROBLEM DESCRIPTION.
C*** IF VARIABLE ND IS REDEFINED TO EXCEED 5 THIS SUBROUTINE MUST
C*** BE MODIFIED
C***

      WRITE (6,1) TITLE
      DO 1 ID=1,2
      WRITE (6,8)
      WRITE (6,9) (TITLE(I),I=1,NITITLE)
      WRITE (6,3) N,NVIR,NI,NF,CRIT,OPCRIT,SQCRIT
      WRITE (6,11) (L(IA),IA=1,30)
      IF (L(6).EQ.1) WRITE (6,12) ATOM(1),ATOM(2)
      WRITE (6,4)
      WRITE (6,5) (ATOM(2*I-1),ATOM(2*I),AMASS(I),RHO(I),RHON(I),ALPHA(I),I=1,NI)
      1 WRITE (6,10) BUCKLE
      WRITE (6,6)
      WRITE (6,7) (IA,(SCALE(IA,IB),IB=1,5),IA=1,NGR)
      WRITE (6,13) (ATOM(1),ATOM(2),I=1,5)
      IB=NG(NVIR-1)
      WRITE (6,14) IF=NG(NVIR-2)
      IF (IB.EQ.0)
      1 IA=1
      WRITE (6,15) (IA,SIGTRB(IA),SIGTRB(IA),SIGEB(IA,1),SIGCB(IA,1),SIGNB(IA,1),SIGEB(IA,1),AMUR(IA,1),IA=1,IB)
      IF (NI.EQ.1) GO TO 2
      WRITE (6,16) (ATOM(3),ATOM(4),I=1,5), (ATOM(5),ATOM(6),I=1,5)
      WRITE (6,16) (IA,(SIGER(IA,IC),SIGCR(IA,IC),SIGNB(IA,IC),SIGFR(IA,IC),AMUR(IA,IC),IC=2,3),IA=1,IB)
      1 IF (NI.LT.4) GO TO 2
      WRITE (6,15) (ATOM(7),ATOM(8),I=1,5), (ATOM(9),ATOM(10),I=1,5)
      WRITE (6,16) (IA,(SIGFR(IA,IC),SIGCR(IA,IC),SIGNB(IA,IC),SIGFR(IA,IC),AMUR(IA,IC),IC=4,5),IA=1,IB)
      1 IC=1
      2 CONTINUE
      2 RETURN
C***
C***
3 FORMAT ('0',//,15X,'NUMBER OF REAL TRANSIENT STATES',I8,/,15X,'NUMBER OF SCAT
1X,'NUMBER OF VIRTUAL TRANSIENT STATES',I8,/,15X,'NUMBER OF FISSION ISOTOPES',
2X,'NUMBER OF ISOTOPES',I8,/,15X,'NUMBER OF FISSION ISOTOPES',I8,/,15X,'
3X,'ARRAY GENERATION CRITERION',F8.4,/,15X,'ARRAY SQUARE
4X,'ARRAY GENERATION CRITERION',F8.4)
5NG CRITERION
4 FORMAT ('0',//,24X,'---SYSTEM PARAMETERS---',//,9X,'ISCTGPE
1 ATOMIC MASS DENSITY NUCLEAR DENSITY ALPHA',//)
5 FORMAT (8X,2A4,F14.3,F16.5,F12.4)
6 FORMAT ('1',//,29X,'---TABLE OF SCALE FACTORS---',//,18X,'GROUP
1 ISOTOPIC INDEX',/,28X,'1
2 5,/,
7 FORMAT ('122,5F8.2)
8 FORMAT ('1',//,/,
1 1,/,
2 2,
5 FORMAT (6X,18A4)
10 FORMAT ('0',//,/,
1)
11 FORMAT ('0',//,/, LOGICALS = '6(512,1X),//) THE ISOTOPE '2A4)
12 FORMAT ('0',//,/, STATE WITH OPTIMIZED FOR THE ISOTOPE '2A4)
13 FORMAT ('1',35X,'---MACROSCOPIC CROSS SECTIONS---',//,/, GROUP
1 1,38X,5(A9,A4),/,
2 ELASTIC CAPTURE INELASTIC TOTAL FISSION LEAKAGE COSINE',//)
3
14 FORMAT (14,F14.6,7F13.6)
15 FORMAT ('1',/,'GROUP',A7,A4,9(A7,A4),/,',', ELASTIC CAPTURE ELASTIC INELASTIC
1 INELASTIC FISSION COSINE',//)
2 INELASTIC FISSION COSINE',//)
16 FORMAT (15,10F11.5)
END

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SUBROUTINE WRITE2 (N)
COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SFLUX(200),RR(200),G(
171),SIGT(71),SIGF(71,5),SIGN(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5),
2,PB2(71),SIGR(71),FIS(400),LEAK(400),CAPT(400),TOTL(400),SDENS(4
300),DENS(400),FLCOR(200),TIM(400),U(201),DU(201),V(201),CV(201),E(
4201),DE(201),EII(201),VII(201),ANUNOR(71),NG(201),FVAR1(204),FVAR2
5(204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),ATCM(10),BGF(26),PGSF(2
16),COMMON(21,6),DEABN(26),DT(21),EABN(26),FMIN(5),ERR(21,2),EXM(21
2,2),H(12,12,5),L(30),NMJM(21),NNG(26),NORM(6),RHON(5),R(6),RFO(5),
3RI(4),RSS(26,6,4,5),SARG(5),SIGTB(26,5),SIGFB(26,5),SINR(26,5),SI
4GER(26,5),SIGEBT(26,5),AMUB(26,5),DB2A(26),SIGTPB(26),SIGIAT(26),SI
5GEP(5),SSTAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),RSC(6),RSCR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCFNER,CONVC,CRIT,CSENER,DCI,DTI,EEFK,ESTK,I,I
1A,IR,IC,ID,IL,IG,IH,IND,IPEAK,J,JJ,JRCW,KROW,LL,MAXR,MAXI,MAXV,N,N
2A,ND,NGR,NF,NIT,NM,NMO,NP1,NP2,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
30P,NVIR,NVM,CPCRIT,SCRIT,SQCRIT,SUMPOP,TIMWID,ZA,ZB,ZC,ZD,ZE
REAL LEAK,NORM
DIMENSION DTIM(400),TIMD(400),DINT(200,6),TINT(200),DIST(400,6
1),EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
1,(FIS,DIST)
C***
C***
C***
C***
WRITE2 LISTS STATE STRUCTURE PARAMETERS AND TESTS THE TRANSFER
COEFFICIENT ARRAY FOR NUMERICAL CONSISTENCY. THAT IS, ELEMENTS
ON A ROW (COLUMN OF TRANSPOSED MATRIX HERE) MUST SUM TO UNITY.
IF (L(3),EQ.0) GO TO 1
STATE PARAMETERS ARE LISTED WHEN BOTTOM STATES OF THE REAL AND
VIRTUAL ARRAYS ARE COINCIDENT.
WRITE (6,11)
WRITE (6,12) (IA,UTIA),DU(IA),E(IA),FII(IA),DE(IA),V(IA),VII(IA),D
1V(IA),RR(IA),NG(IA),IA,IA=1,NVIR)
1CONTINUE
TEST ARRAY. ACCEPTABLE TOLERANCE HAS BEEN SET ARBITRARILY AT
+ OR - 0.0001.
DO 2 IA=1,NP3
DO 2 IB=IA,NP3
P(1,IA) IS TEMPORARILY USED TO ACCUMULATE SUMS FOR MATRIX ROW IA.
P(1,IA)=P(1,IA)+P(1B,IA)
ID=2
IF (L(1),EQ.1,AND,L(12),EQ.0,AND,L(2),EQ.1) ID=1
DO 3 IA=ID,NP3
IF (P(1,IA),GT.1.0001) GO TO 4
IF (P(1,IA),LT.0.9999) GO TO 4
1CONTINUE
GO TO 5
ID=NTOP+1
WRITE (6,9) ID
IF ROW SUMS DO NOT LIE WITHIN STATED LIMITS THE PROBLEM IS
TERMINATED. EXCEPTION--IF (N,2N) REACTIONS OCCUR THE ROW SUMS ARE
LISTED BUT EXECUTION IS ALLOWED TO CONTINUE
WRITE (6,10) (P(1,IA),IA=1,NP3)
IF (L(19),EQ.1) GO TO 7
GO TO 8
DO 6 IA=1,NP3
P(1,IA)=0.
8RETURN
C***
9FORMAT (1,'!-----TRANSFER COEFFICIENT ROW SUM CRITERIA IS NOT SAT
11SIFIED-----',///,' ROW SUMS, BEGINNING WITH ROW',I4,///)
10FORMAT (10F12.8)
11FORMAT (1,'!,,,40X,'-----STATE PARAMETERS-----',///,' STATE U
1DU
2DV
NG STATE',//)

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*** XSECT ***

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SUBROUTINE XSECT
COMMON P(74,74),POP(74,2),PTP(204),FLUX(200),SFLUX(200),RR(200),G(
171),SIGT(71),SIGF(71,5),SIGC(71,5),SIGE(71,5),AMU(71,5)
2,CB2(71),SIGTP(71),LEAK(400),EAPT(400),TOTL(400),SCENS(4
300),DFNS(400),FLCOR(200),TIM(400),U(201),V(201),CV(201),F(
4201),DET(201),EIT(201),VIT(201),ANUNOR(71),NG(201),FVAR1(204),FVAR2
5(204),FVAR3(204),FVAR4(204)
COMMON ALP(5),ALPHA(5),AMASS(5),AMQM(21,6),ATOM(10),RGF(26),PGSF(2
16),CORMQM(21,6),DEABN(26),DT(21),EABN(26),EMIN(5),ERR(21,2),EXW(21
2,2),H(12,12,5),L(30),NMQM(21),NNG(26),NORM(6),RHON(5),R(6),RFD(5),
3RI(4),RSS(26,6,4,5),SARG(5),SIGTR(26,5),SIGFB(26,5),SIGNB(26,5),SI
4GCR(26,5),SIGEB(26,5),AMUB(26,5),DB2R(26),SIGTRB(26),SIGTRT(26),SI
5GEP(5),SSTAR(5),SZERO(5),SCALE(26,5),TITLE(72),ALAM(5),RSC(6),PSCR
6(2),RSDX(2)
COMMON ACCUM,BUCKLE,CCENER,CONVC,CRIT,CSENER,DOIT,DTI,EFFK,ESTK,I,I
1A,IB,IC,ID,IE,IG,IH,IND,IPEAK,J,JJ,JROW,KROW,LL,MAXR,MAXI,MAXV,N,N
2A,NO,NGR,NF,NI,NIT,NM,NMO,NP1,NP2,NP3,NSCAT,NT,NTIME,NTITLE,NTM,NT
3OP,NVIR,NVM,OPCRIT,SCRIT,SQCRIT,SUMPOP,TIMWID,ZA,ZB,ZC,ZE
REAL LEAK,NORM
DIMENSION DTIM(400),TIMD(400),DINT(200,6),TINT(200),CIST(400,6)
1)
DIMENSION N2N(5)
REAL N2N
EQUIVALENCE (P,DTIM), (P(401),TIMD), (P(801),DINT), (P(3201),TINT)
1,(FISS,CIST)
C***
C*** XSECT PERFORMS TWO MANIPULATIONS ON DATA. (1) MACROSCOPIC CROSS
C*** SECTIONS ARE DIVIDED BY THE TOTAL CROSS SECTION, (2) THE INELASTIC
C*** SCATTERING MATRIX IS TRANSPOSED AND NORMALIZED.
C***
DO 2 IA=1,NM
DO 1 IB=1,NI
ZA=1./SIGT(IA)
SIGC(IA,IB)=SIGC(IA,IB)*ZA
SIGN(IA,IB)=SIGN(IA,IB)*ZA
SIGF(IA,IB)=SIGF(IA,IB)*ZA
C*** THE PL ANISOTROPY CORRECTION IS INCLUDED IN ELASTIC CROSS SECTIONS
1 SIGE(IA,IB)=(1.+2./3.*AMASS(IB))-AMU(IA,IB)*SIGE(IA,IB)*ZA
2 CB2(IA)=DB2(IA)*ZA
C*** IF THE INELASTIC ARRAY HAS BEEN NORMALIZED AND TRANSPOSED THE
C*** FOLLOWING IS BYPASSED.
C*** IF (L(2).EQ.0) GO TO 14
C*** IF THERE ARE NO INELASTIC CROSS SECTIONS IN THE DATA SET THE
C*** NORMALIZING ROUTINE IS BYPASSED.
C*** TEST FOR INELASTIC SCATTERING
DO 3 IA=1,NI
DO 3 IB=1,26
IF (SIGNB(IB,IA).EQ.0.) GO TO 3
L(12)=1
GO TO 4
3 CONTINUE
4 CONTINUE
IF (L(12).EQ.0) GO TO 14
MICROSCOPIC TRANSFER CROSS SECTIONS ARE NORMALIZED TO UNIT KOW
C*** SUM BELOW.
DO 9 IA=1,NI
C*** RATIO OF NUMBER OF NEUTRONS LEAVING AN INELASTIC SCATTERING
C*** REACTION TO NUMBER ENTERING IS EVALUATED. (N,2N) REACTIONS ARE
C*** IMPORTANT ONLY IN GROUP 1.
ANOR=0.
N2N(IA)=1.
DO 5 IC=1,12

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5 ANOR=ANOR+H(1,IC,IA)
  ZB=SIGNR(1,IA)/RHCN(IA)
  IF (ANOR.EQ.ZB) GO TO 6
  N2N(IA)=ANOR/ZB
  L(19)=1
  WRITE (6,15) ATOM(2*IA-1),ATOM(2*IA)
6 CONTINUE
  DO 9 IB=1,12
  ANOR=0
  DO 7 IC=1,12
  ANOR=ANOR+H(IB,IC,IA)
  IF (ANOR.EQ.0.) GO TO 9
  ANOR=1./ANOR
  DO 8 IC=1,12
  H(IB,IC,IA)=ANOR*H(IB,IC,IA)
7 CONTINUE
  SHIFT ROWS OF H-ARRAY TO PLACE ELEMENTS IN PROPER POSITION FOR
  THIS PROGRAM.
  DO 11 IA=1,N1
  IE=0
  DO 10 IB=2,12
  IE=IE+1
  IC=IB-1
  IC=IB,12
  IC=12-IC+IB
  H(IB,ID,IA)=H(IB,ID-IE,IA)
  DO 11 IB=2,12
  ID=IB-1
  ID=IB,1
  DO 11 IC=1,ID
  H(IB,IC,IA)=0.
11 CONTINUE
  DO 12 IA=1,N1
  DO 12 IB=1,12
  H(1,IB,IA)=N2N(IA)*H(1,IB,IA)
  WRITE (6,16)
  DO 13 I=1,N1
  WRITE (6,17) ATOM(2*I-1),ATOM(2*I),N2N(I)
  WRITE (6,18) (IA,(H(IA,IB,I),IB=1,12),IA=1,12)
13 CONTINUE
14 RETURN
C***
C***
15 FORMAT ('0', ' *** (N,2N) REACTIONS OCCUR IN ',2A4,'. RCW SUM ERR
16S WILL NOT STOP EXECUTION OF THIS PROGRAM,')
16 FORMAT ('1',4X,'-----NORMALIZED TRANSFER COEFFICIENTS FOR INELASTI
1C SCATTERING-----',//,34X,' H(1,J),',, 1 9,30X,' J',/,9
2 10 11 12')
17 FORMAT ('0',/, '-----',2A4,'-----',/, ' NEUTRON YIELD FROM INELASTI
1C SCATTERING IN GROUP 1 =',F5.2,/)
18 FORMAT (13,F9.3,11F7.3)
  ENCL

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APPENDIX G
SLOAD Listing with output

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(C ** SAMPLE CROSS SECTION SET FOR INPUT TO SLOAD

UP ANILIN-235

235	6.3000	1.7500	0.0200	1.0300	3.5000	C.8400
1	7.4000	1.1500	0.0300	1.9200	4.3000	C.8000
2	7.7000	1.2500	0.0400	1.9100	4.5000	0.7100
3	7.0000	1.2800	0.0600	1.7600	3.9000	0.5500
4	6.6000	1.2500	0.1200	1.3800	3.8500	0.4500
5	7.4000	1.2300	0.1700	1.2000	4.8000	C.3500
6	9.2000	1.4100	0.2500	1.0000	6.5400	0.2300
7	11.2000	1.7000	0.4000	0.6000	8.5000	0.1300
8	12.5000	2.1000	0.6000	0.1800	9.6200	C.0700
9	14.0000	2.6500	1.0000	0.0600	10.3000	0.0400
10	16.0000	3.4000	1.5000	0.0000	11.1000	0.0200
11	19.0000	4.4000	2.1000	0.0000	12.5000	0.0100
12	23.0000	5.4000	2.7500	0.0000	14.8000	0.0027
13	27.0000	7.3000	3.8000	0.0000	15.9000	0.0027
14	32.0000	11.0000	6.3000	0.0000	14.7000	0.0027
15	38.0000	16.0000	9.5000	0.0000	12.5000	0.0027
16	47.0000	22.0000	13.5000	0.0000	12.2000	0.0027
17	69.0000	35.0000	22.0000	0.0000	12.0000	0.0027
18	88.0000	45.0000	31.0000	0.0000	12.0000	0.0027
19	111.0000	45.0000	54.0000	0.0000	12.0000	0.0027
20	93.0000	37.0000	44.0000	0.0000	12.0000	0.0027
21	39.0000	20.0000	7.0000	0.0000	12.0000	0.0027
22	61.0000	35.0000	13.0000	0.0000	13.0000	0.0027
23	48.0000	64.0000	10.0000	0.0000	14.0000	0.0027
24	205.0000	155.0000	35.0000	0.0000	15.0000	0.0027
25	698.0000	582.0000	101.0000	0.0000	15.0000	0.0027

Wavelength	Intensity	Wavelength	Intensity	Wavelength	Intensity	Wavelength	Intensity
235	0.01	25	0.43	35	0.14	0.01	
236	0.04	51	0.51	11	0.04	0.01	
237	0.04	54	0.30	12	0.01		
238	0.04	55	0.30	12	0.01		
239	0.04	56	0.30	12	0.01		
240	0.04	57	0.30	12	0.01		
241	0.04	58	0.30	12	0.01		
242	0.04	59	0.30	12	0.01		
243	0.04	60	0.30	12	0.01		
244	0.04	61	0.30	12	0.01		
245	0.04	62	0.30	12	0.01		
246	0.04	63	0.30	12	0.01		
247	0.04	64	0.30	12	0.01		
248	0.04	65	0.30	12	0.01		
249	0.04	66	0.30	12	0.01		
250	0.04	67	0.30	12	0.01		
251	0.04	68	0.30	12	0.01		
252	0.04	69	0.30	12	0.01		
253	0.04	70	0.30	12	0.01		
254	0.04	71	0.30	12	0.01		
255	0.04	72	0.30	12	0.01		
256	0.04	73	0.30	12	0.01		
257	0.04	74	0.30	12	0.01		
258	0.04	75	0.30	12	0.01		
259	0.04	76	0.30	12	0.01		
260	0.04	77	0.30	12	0.01		
261	0.04	78	0.30	12	0.01		
262	0.04	79	0.30	12	0.01		
263	0.04	80	0.30	12	0.01		
264	0.04	81	0.30	12	0.01		
265	0.04	82	0.30	12	0.01		
266	0.04	83	0.30	12	0.01		
267	0.04	84	0.30	12	0.01		
268	0.04	85	0.30	12	0.01		
269	0.04	86	0.30	12	0.01		
270	0.04	87	0.30	12	0.01		
271	0.04	88	0.30	12	0.01		
272	0.04	89	0.30	12	0.01		
273	0.04	90	0.30	12	0.01		
274	0.04	91	0.30	12	0.01		
275	0.04	92	0.30	12	0.01		
276	0.04	93	0.30	12	0.01		
277	0.04	94	0.30	12	0.01		
278	0.04	95	0.30	12	0.01		
279	0.04	96	0.30	12	0.01		
280	0.04	97	0.30	12	0.01		
281	0.04	98	0.30	12	0.01		
282	0.04	99	0.30	12	0.01		
283	0.04	100	0.30	12	0.01		

I	DU	SIGT	SIGF	SIGC	SIGN	SIGE	AMU	I
1	0.48	6.30	1.75	0.020	1.03	3.50	0.84	1
2	0.48	7.40	1.15	0.030	1.92	4.30	0.80	2
3	0.48	7.70	1.25	0.040	1.91	4.50	0.71	3
4	0.57	7.00	1.28	0.060	1.76	3.90	0.55	4
5	0.57	6.60	1.25	0.120	1.38	3.85	0.45	5
6	0.69	7.40	1.23	0.170	1.20	4.80	0.35	6
7	0.69	9.20	1.41	0.250	1.00	6.54	0.23	7
8	0.69	11.20	1.70	0.400	0.60	8.50	0.13	8
9	0.77	12.50	2.10	0.600	0.18	9.62	0.07	9
10	0.77	14.00	2.65	1.000	0.06	10.30	0.04	10
11	0.77	16.00	3.40	1.500	0.0	11.10	0.02	11
12	0.77	19.00	4.40	2.100	0.0	12.50	0.01	12
13	0.77	23.00	5.40	2.750	0.0	14.80	0.00	13
14	0.77	27.00	7.30	3.800	0.0	15.90	0.00	14
15	0.77	32.00	11.00	6.300	0.0	14.70	0.00	15
16	0.77	38.00	16.00	9.500	0.0	12.50	0.00	16
17	0.77	47.70	22.00	13.500	0.0	12.20	0.00	17
18	0.77	69.00	35.00	22.000	0.0	12.00	0.00	18
19	0.77	88.00	45.00	31.000	0.0	12.00	0.00	19
20	0.77	111.00	45.00	54.000	0.0	12.00	0.00	20
21	0.77	93.00	37.00	44.000	0.0	12.00	0.00	21
22	0.77	39.00	20.00	7.000	0.0	12.00	0.00	22
23	0.77	61.00	35.00	13.000	0.0	13.00	0.00	23
24	0.77	88.00	64.00	10.000	0.0	14.00	0.00	24
25	0.77	205.00	155.00	35.000	0.0	15.00	0.00	25
26	0.0	698.00	582.00	101.000	0.0	15.00	0.00	26

RESONANCE SELF SHIELDING TABLE FOR ISOTOPE 34 - U-235 .

GROUP	FISSION			CAPTURE			TOTAL			ELASTIC			
	1	2	3	4	5	6	1	2	3	4	5	6	
1	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
2	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
3	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
4	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
5	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
6	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
7	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
8	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
9	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
10	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
11	0.97	0.97	0.98	1.00	0.97	0.98	1.00	0.98	0.98	0.99	1.00	1.00	1.00
12	0.94	0.95	0.96	0.99	0.94	0.95	0.96	0.96	0.97	0.98	1.00	1.00	1.00
13	0.89	0.91	0.93	0.99	0.89	0.91	0.93	0.93	0.94	0.96	0.99	1.00	1.00
14	0.82	0.85	0.88	0.97	0.82	0.85	0.88	0.88	0.90	0.92	0.96	1.00	1.00
15	0.75	0.78	0.82	0.94	0.74	0.77	0.81	0.82	0.85	0.88	0.93	1.00	1.00
16	0.67	0.71	0.75	0.90	0.65	0.69	0.74	0.73	0.75	0.78	0.85	1.00	1.00
17	0.62	0.63	0.67	0.83	0.56	0.60	0.64	0.62	0.66	0.70	0.80	1.00	1.00
18	0.55	0.58	0.61	0.77	0.48	0.52	0.56	0.50	0.53	0.56	0.70	1.00	1.00
19	0.52	0.54	0.56	0.70	0.43	0.45	0.48	0.39	0.40	0.42	0.57	1.00	1.00
20	0.49	0.50	0.52	0.64	0.38	0.40	0.42	0.32	0.34	0.36	0.42	1.00	1.00
21	0.47	0.48	0.50	0.62	0.35	0.37	0.40	0.29	0.30	0.31	0.42	1.00	1.00
22	0.74	0.76	0.79	0.90	0.56	0.59	0.62	0.64	0.66	0.68	0.82	1.00	1.00
23	0.70	0.72	0.74	0.87	0.64	0.66	0.68	0.63	0.64	0.65	0.80	1.00	1.00
24	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
25	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00
26	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00	1.00

INELASTIC SCATTERING MATRIX FOR ISOTOPE NUMBER 34 - U-235 .

N	0	1	2	3	4	5	6	7	8	9	10	11
1	0.0	0.01	0.05	0.25	0.43	0.56	0.35	0.14	0.04	0.01	0.0	0.0
2	0.02	0.08	0.35	0.54	0.51	0.26	0.11	0.04	0.01	0.0	0.0	0.0
3	0.10	0.27	0.53	0.54	0.30	0.12	0.04	0.01	0.0	0.0	0.0	0.0
4	0.20	0.35	0.57	0.40	0.16	0.06	0.02	0.0	0.0	0.0	0.0	0.0
5	0.20	0.51	0.37	0.20	0.08	0.02	0.0	0.0	0.0	0.0	0.0	0.0
6	0.44	0.44	0.22	0.08	0.02	0.0	0.0	0.0	0.0	0.0	0.0	0.0
7	0.61	0.38	0.01	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
8	0.21	0.29	0.08	0.02	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
9	0.09	0.07	0.02	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
10	0.05	0.01	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

APPENDIX H

EXAMPLES OF OUTPUT FROM MOD-5

Note: The listings that follow are incomplete and are intended only to provide verification that the sample data decks have been properly executed.

STATE WIDTH OPTIMIZATION DATA

THE INITIAL LETHARGY DECREMENT (LOG10) IS 0.96645E-01

1 STATE(S) PER SCATTERING INTERVAL

THE CONVERGENCE CRITERIA IS 0.00001

AT ITERATION NUMBER 1, THE ERROR IS -0.02748, RR(2) = 1.49646, AND THE NEW LETHARGY WIDTH IS 0.10468E 00

AT ITERATION NUMBER 2, THE ERROR IS -0.00554, RR(2) = 1.54896, AND THE NEW LETHARGY WIDTH IS 0.10628E 00

AT ITERATION NUMBER 3, THE ERROR IS -0.00116, RR(2) = 1.55972, AND THE NEW LETHARGY WIDTH IS 0.10662E 00

AT ITERATION NUMBER 4, THE ERROR IS -0.00025, RR(2) = 1.56198, AND THE NEW LETHARGY WIDTH IS 0.10669E 00

AT ITERATION NUMBER 5, THE ERROR IS -0.00005, RR(2) = 1.56246, AND THE NEW LETHARGY WIDTH IS 0.10671E 00

AT ITERATION NUMBER 6, THE ERROR IS -0.00001, RR(2) = 1.56256, AND THE NEW LETHARGY WIDTH IS 0.10671E 00

AT ITERATION NUMBER 7, THE ERROR IS -0.00000, RR(2) = 1.56258, AND THE NEW LETHARGY WIDTH IS 0.10671E 00

 * MOD-5 *

EXAMPLE NUMBER 1
 EVALUATION OF ASYMPTOTIC SOLUTION TO SLOWING DOWN EQUATION.
 INFINITE GRAPHITE MODERATOR. DELTA SOURCE IN TIME AND ENERGY.

NUMBER OF REAL TRANSIENT STATES 51
 NUMBER OF VIRTUAL TRANSIENT STATES 51
 NUMBER OF SCATTERING ISOTOPES 1
 NUMBER OF FISSION ISOTOPES 0
 ROW GENERATION CRITERION 2.0010
 ARRAY GENERATION CRITERION 0.4000
 ARRAY SQUARING CRITERION 0.2000

LOGICALS = 0 1 1 1 0 1 1 0 0 0 1 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0

STATE WIDTH OPTIMIZED FOR THE ISOTOPE CARBON

-----SYSTEM PARAMETERS-----				
ISOTOPE	ATOMIC MASS	MASS DENSITY	NUCLEAR DENSITY	ALPHA
CARBON	12.011	1.6400	0.08224	0.7162

SYSTEM BUCKLING = 0.0 /SQ.CM.

-----STATE PARAMETERS-----

STATE	U	DU	F	EII	DE	V	VII	DV	RR	NG	STATE
1	0.457E-01	65F-01	694E	05	0.90E	04	08	0.581E	0.03E	8	1
2	0.914E-01	657E-01	625E	05	0.686E	04	08	0.820E	0.03E	8	2
3	0.371E-01	457E-01	336E	05	0.257E	04	08	0.147E	0.03E	8	3
4	0.229E-01	457E-01	957E	04	0.381E	04	08	0.552E	0.03E	0	4
5	0.474E-01	457E-01	833E	04	0.087E	04	08	0.026E	0.03E	0	5
6	0.720E-01	457E-01	372E	04	0.447E	04	08	0.560E	0.03E	0	6
7	0.170E-01	457E-01	855E	04	0.607E	04	08	0.149E	0.03E	0	7
8	0.215E-01	457E-01	451E	04	0.168E	04	08	0.785E	0.03E	1	8
9	0.949E-01	457E-01	135E	03	0.422E	03	08	0.926E	0.03E	1	9
10	0.194E-01	457E-01	777E	03	0.161E	03	08	0.178E	0.03E	1	10
11	0.686E-01	457E-01	431E	03	0.933E	03	08	0.507E	0.03E	1	11
12	0.368E-01	457E-01	222E	03	0.113E	03	08	0.332E	0.03E	1	12
13	0.931E-01	457E-01	039E	03	0.254E	02	07	0.422E	0.03E	1	13
14	0.423E-01	457E-01	243E	03	0.628E	02	07	0.217E	0.03E	1	14
15	0.914E-01	457E-01	606E	03	0.709E	02	07	0.152E	0.03E	1	15
16	0.651E-01	457E-01	533E	03	0.119E	02	07	0.206E	0.03E	1	16
17	0.897E-01	457E-01	647E	02	0.596E	02	07	0.376E	0.03E	1	17
18	0.388E-01	457E-01	847E	02	0.219E	02	07	0.987E	0.03E	0	18
19	0.663E-01	457E-01	742E	02	0.921E	02	07	0.410E	0.03E	0	19
20	0.126E-01	457E-01	242E	02	0.205E	01	07	0.386E	0.03E	0	20
21	0.371E-01	457E-01	365E	02	0.894E	01	07	0.110E	0.03E	0	21
22	0.663E-01	457E-01	517E	02	0.677E	01	07	0.851E	0.03E	0	22
23	0.126E-01	457E-01	742E	02	0.321E	01	07	0.651E	0.03E	0	23
24	0.371E-01	457E-01	987E	01	0.421E	01	07	0.460E	0.03E	0	24
25	0.508E-01	457E-01	518E	01	0.186E	01	07	0.291E	0.03E	0	25
26	0.846E-01	457E-01	139E	01	0.116E	01	07	0.140E	0.03E	0	26
27	0.609E-01	457E-01	908E	01	0.794E	00	06	0.937E	0.03E	0	27
28	0.337E-01	457E-01	492E	01	0.354E	00	06	0.841E	0.03E	0	28
29	0.582E-01	457E-01	168E	01	0.252E	00	06	0.729E	0.03E	0	29
30	0.032E-01	457E-01	120E	00	0.989E	00	06	0.463E	0.03E	0	30
31	0.058E-01	457E-01	568E	00	0.155E	00	06	0.831E	0.03E	0	31
32	0.058E-01	457E-01	347E	00	0.215E	00	06	0.279E	0.03E	0	32
33	0.058E-01	457E-01	690E	00	0.542E	00	06	0.342E	0.03E	0	33
34	0.058E-01	457E-01	170E	00	0.851E	00	06	0.356E	0.03E	0	34
35	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	35
36	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	36
37	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	37
38	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	38
39	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	39
40	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	40
41	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	41
42	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	42
43	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	43
44	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	44
45	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	45
46	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	46
47	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	47
48	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	48
49	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	49
50	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	50
51	0.058E-01	457E-01	635E	00	0.561E	00	06	0.612E	0.03E	0	51

-----SOURCE VECTOR PCP(1,1) (NEUTRONS PER STATE)-----

GENERATING ROUTINE IS BEING CALLED FOR 8 ITERATIONS BEGINNING AT TIME DOT = 7.634E-12 SECONDS.
DOT IS DEFINED FOR VIRTUAL STATE 2 FOR WHICH THE DECAY CONSTANT IS 2.055E 08.

---AT OPERATING ITERATION NO. 1 AND TIME = 0.0 + 0.0 SECONDS,
THE NEUTRON DENSITY (LETHARGY) VECTOR IS WRITTEN BEGINNING WITH VIRTUAL STATE 1.

[illegible]

110

MEAN ENERGY (DENSITY AVERAGED)	=	1.6936E 05	MEAN VELOCITY	=	5.6923E 08
R.M.S. ENERGY	=	1.6936E 05	R.M.S. VELOCITY	=	5.6923E 08
RELATIVE STANDARD DEVIATION	=	0.0	RELATIVE STANDARD DEVIATION	=	0.0

```

NT= 1 NTMF= 1 IPEAK= 2 ZA= 0.400E 00 ZX= 0.0

```

---AT OPERATING ITERATION NO. 2 AND TIME = 0.0 + 0.195E-08 SECONDS,
THE NEUTRON DENSITY (LETHARGY) VECTOR IS WRITTEN BEGINNING WITH VIRTUAL STATE 1.

[illegible]

ABSORBING STATE POPULATIONS	0.0	0.0	0.0
0.0	0.0	0.0	0.0

MEAN ENERGY (DENSITY AVERAGED)	=	1.5777E 05	MEAN VELOCITY	=	5.4844E J8
R.M.S. ENERGY	=	1.5898E J5	R.M.S. VELOCITY	=	5.4853E J8
RELATIVE STANDARD DEVIATION	=	1.1848E -01	RELATIVE STANDARD DEVIATION	=	6.2966E -02

```

NT= 2  NTIME= 1  IPEAK= 2  ZA= 0.400E 00  ZX= 0.0

```

5.4844E-08
5.4953E-08
6.2964E-02

----- A MOD5 CALCULATION -----

EXAMPLE NUMBER 1
EVALUATION OF ASYMPTOTIC SOLUTION TO SLOWING DOWN EQUATION.
INFINITE GRAPHITE MODERATOR. DELTA SOURCE IN TIME AND ENERGY.

THE CENTRAL CORE FLUX-AVERAGED ENERGY IS 1.297E 04 EV.
THE LEAKAGE FLUX-AVERAGED ENERGY IS 0.0 EV.

AT	6.215E-05 SECONDS--	THE NEUTRONS HAVE SUFFERED NON-FISSION CAPTURE,
0.0	PERCENT OF THE NEUTRONS HAVE LEAKED FROM THE SYSTEM,	
0.0	PERCENT OF THE NEUTRONS HAVE CAUSED FISSION,	
99.647	PERCENT OF THE NEUTRONS HAVE ENERGY BELOW 1.00E 00 EV,	
0.001	PERCENT ARE STILL SLOWING DOWN,	
0.352	PERCENT HAVE LOST FROM THE SYSTEM, AND	
0.0	PERCENT HAVE BEEN BYPASSED BY THE ARRAY.	

***** FINAL STATE VECTOR

Case	Time	Iterations	Iterations per second	Iterations per second (approx.)
1	0.0	0.0	0.0	0.0
2	0.0	0.0	0.0	0.0
3	0.0	0.0	0.0	0.0
4	0.0	0.0	0.0	0.0
5	0.0	0.0	0.0	0.0
6	0.0	0.0	0.0	0.0
7	0.0	0.0	0.0	0.0
8	0.0	0.0	0.0	0.0
9	0.0	0.0	0.0	0.0
10	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0
13	0.0	0.0	0.0	0.0
14	0.0	0.0	0.0	0.0
15	0.0	0.0	0.0	0.0
16	0.0	0.0	0.0	0.0
17	0.0	0.0	0.0	0.0
18	0.0	0.0	0.0	0.0
19	0.0	0.0	0.0	0.0
20	0.0	0.0	0.0	0.0
21	0.0	0.0	0.0	0.0
22	0.0	0.0	0.0	0.0
23	0.0	0.0	0.0	0.0
24	0.0	0.0	0.0	0.0
25	0.0	0.0	0.0	0.0
26	0.0	0.0	0.0	0.0
27	0.0	0.0	0.0	0.0
28	0.0	0.0	0.0	0.0
29	0.0	0.0	0.0	0.0
30	0.0	0.0	0.0	0.0
31	0.0	0.0	0.0	0.0
32	0.0	0.0	0.0	0.0
33	0.0	0.0	0.0	0.0
34	0.0	0.0	0.0	0.0
35	0.0	0.0	0.0	0.0
36	0.0	0.0	0.0	0.0
37	0.0	0.0	0.0	0.0
38	0.0	0.0	0.0	0.0
39	0.0	0.0	0.0	0.0
40	0.0	0.0	0.0	0.0
41	0.0	0.0	0.0	0.0
42	0.0	0.0	0.0	0.0
43	0.0	0.0	0.0	0.0
44	0.0	0.0	0.0	0.0
45	0.0	0.0	0.0	0.0
46	0.0	0.0	0.0	0.0
47	0.0	0.0	0.0	0.0
48	0.0	0.0	0.0	0.0
49	0.0	0.0	0.0	0.0
50	0.0	0.0	0.0	0.0
51	0.0	0.0	0.0	0.0
52	0.0	0.0	0.0	0.0
53	0.0	0.0	0.0	0.0
54	0.0	0.0	0.0	0.0
55	0.0	0.0	0.0	0.0
56	0.0	0.0	0.0	0.0
57	0.0	0.0	0.0	0.0
58	0.0	0.0	0.0	0.0
59	0.0	0.0	0.0	0.0
60	0.0	0.0	0.0	0.0
61	0.0	0.0	0.0	0.0
62	0.0	0.0	0.0	0.0
63	0.0	0.0	0.0	0.0
64	0.0	0.0	0.0	0.0
65	0.0	0.0	0.0	0.0
66	0.0	0.0	0.0	0.0
67	0.0	0.0	0.0	0.0
68	0.0	0.0	0.0	0.0
69	0.0	0.0	0.0	0.0
70	0.0	0.0	0.0	0.0
71	0.0	0.0	0.0	0.0
72	0.0	0.0	0.0	0.0
73	0.0	0.0	0.0	0.0
74	0.0	0.0	0.0	0.0
75	0.0	0.0	0.0	0.0
76	0.0	0.0	0.0	0.0
77	0.0	0.0	0.0	0.0
78	0.0	0.0	0.0	0.0
79	0.0	0.0	0.0	0.0
80				

***** FINAL CORE FLUX VECTOR

[illegible]

***** FINAL SURFACE FLUX VECTOR

TABLE OF MOMENTS OF DISTRIBUTIONS
(UNITS ARE MICROSECONDS)

NEUTRON DENSITY AT 1.114E 00 EV.					SLOWING DOWN DENSITY AT 1.000E 00 EV.				
N	CALC.	ASYM.	T.C.	ERROR	N	CALC.	ASYM.	T.C.	ERROR
1	2.4036E 01	2.3702E 01	5.82E-04	1.410	1	2.3900E 01	2.3677E 01	5.09E-04	0.939
2	6.0858E 02	5.9167E 02	5.10E-02	2.857	2	6.0639E 02	5.9145E 02	4.49E-02	2.526
3	1.6347E 04	1.5533E 04	3.63E 00	5.239	3	1.6312E 04	0.0	3.19E 00	0.0
4	4.6519E 05	4.2826E 05	2.43E 02	8.624	4	4.6466E 05	0.0	2.14E 02	0.0
5	1.4004E 07	1.2384E 07	1.63E 04	13.101	5	1.3998E 07	0.0	1.41E 04	0.0
-1	4.5001E-02	4.4500E-02	0.0	1.125	-1	4.4474E-02	0.0	0.0	0.0
-2	2.1379E-03	2.0919E-03	0.0	2.201	-2	2.1053E-03	0.0	0.0	0.0
-3	1.0823E-04	1.0403E-04	0.0	4.033	-3	1.0621E-04	0.0	0.0	0.0
RSD	2.3101E-01	2.3065E-01		0.158		2.4824E-01	2.3453E-01		5.848

CALC. = CALCULATED FROM DISTRIBUTION AND CORRECTED FOR TRUNCATION ERROR
 ASYM. = ERIKSSON'S ASYMPTOTIC MOMENTS
 T.C. = TRUNCATION CORRECTION
 ERROR = PERCENT DEVIATION OF CALCULATED FROM ASYMPTOTIC
 RSD = RELATIVE STANDARD DEVIATION

**** RATIO OF COMPUTED TO EXACT MOMENTS OF DENSITY

N	CCRMOM/EXM
1	1.01410389
2	1.02857208
3	1.05239391
4	1.08623600
5	1.13100910
-1	1.01125050
-2	1.02201271
-3	1.04032803

INTERPOLATED DISTRIBUTIONS
DTI = 5.0E-07 SECONDS

N	TIME	DENSITY	S.D.D.	N
1	250E-07	5.126E-61	172E-61	1
2	500E-07	9.440E-47	181E-47	2
3	750E-07	5.341E-39	3.635E-39	3
4	100E-07	9.142E-34	8.771E-34	4
5	250E-07	7.121E-30	5.552E-30	5
6	500E-07	8.286E-24	7.454E-24	6
7	750E-07	3.750E-20	3.344E-20	7
8	100E-06	2.279E-17	2.079E-17	8
9	125E-06	1.924E-16	1.801E-16	9
10	150E-06	1.212E-15	1.032E-15	10
11	175E-06	3.403E-14	3.831E-14	11
12	200E-06	3.216E-13	3.805E-13	12
13	225E-06	2.339E-12	3.079E-12	13
14	250E-06	1.787E-11	1.087E-11	14
15	275E-06	6.049E-10	5.517E-10	15
16	300E-06	3.123E-09	2.026E-09	16
17	325E-06	1.447E-07	1.242E-07	17
18	350E-06	1.231E-06	1.481E-06	18
19	375E-06	1.362E-05	1.816E-05	19
20	400E-06	1.706E-04	1.507E-04	20
21	425E-06	1.439E-03	1.888E-03	21
22	450E-06	8.898E-02	1.056E-02	22
23	475E-06	4.288E-01	5.555E-01	23
24	500E-06	1.685E-00	2.000E-00	24
25	525E-06	1.594E-01	1.507E-01	25
26	550E-06	1.611E-01	1.051E-01	26
27	575E-06	4.165E-01	3.155E-01	27
28	600E-06	9.553E-01	1.810E-01	28
29	625E-06	2.891E-02	1.959E-02	29
30	650E-06	7.056E-03	9.200E-03	30
31	675E-06	1.204E-03	1.830E-03	31
32	700E-06	1.948E-03	1.916E-03	32
33	725E-06	3.004E-03	1.958E-03	33
34	750E-06	4.303E-03	3.740E-03	34
35	775E-06	6.303E-03	6.210E-03	35
36	800E-06	8.645E-04	8.542E-04	36
37	825E-06	1.149E-04	1.368E-04	37
38	850E-06	1.483E-04	1.468E-04	38
39	875E-06	1.864E-04	1.846E-04	39
40	900E-06	2.287E-04	2.670E-04	40
41	925E-06	2.743E-04	3.220E-04	41
42	950E-06	3.225E-04	3.720E-04	42
43	975E-06	3.720E-04	4.067E-04	43
44	1000E-06	4.067E-04	4.555E-04	44

INTERPOLATED DISTRIBUTIONS
DTI = 5.0E-07 SECONDS

N	TIME	DENSITY	S.D.D.	N
101	4.300E-05	1.49E	451E	101
102	4.350E-05	1.48E	459E	102
103	4.400E-05	1.41E	1.209E	103
104	4.450E-05	8.13E	9.436E	104
105	4.500E-05	4.13E	7.156E	105
106	4.550E-05	6.05E	6.075E	106
107	4.600E-05	5.21E	5.235E	107
108	4.650E-05	4.79E	4.877E	108
109	4.700E-05	3.86E	3.332E	109
110	4.750E-05	3.20E	2.861E	110
111	4.800E-05	2.85E	2.455E	111
112	4.850E-05	2.44E	2.105E	112
113	4.900E-05	2.07E	1.802E	113
114	4.950E-05	1.53E	1.543E	114
115	5.000E-05	1.31E	1.319E	115
116	5.050E-05	1.12E	1.128E	116
117	5.100E-05	1.12E	1.127E	117
118	5.150E-05	9.18E	8.224E	118
119	5.200E-05	6.59E	6.081E	119
120	5.250E-05	5.95E	5.134E	120
121	5.300E-05	5.07E	4.900E	121
122	5.350E-05	4.31E	4.690E	122
123	5.400E-05	3.67E	4.330E	123
124	5.450E-05	3.26E	3.968E	124
125	5.500E-05	2.58E	3.268E	125
126	5.550E-05	2.18E	2.929E	126
127	5.600E-05	1.62E	2.680E	127
128	5.650E-05	1.31E	2.380E	128
129	5.700E-05	1.17E	2.168E	129
130	5.750E-05	9.30E	1.936E	130
131	5.800E-05	8.41E	1.807E	131
132	5.850E-05	7.12E	1.667E	132
133	5.900E-05	6.03E	1.536E	133
134	5.950E-05	5.18E	1.407E	134
135	6.000E-05	4.43E	1.285E	135
136	6.050E-05	3.66E	1.168E	136
137	6.100E-05	3.13E	1.057E	137
138	6.150E-05	2.62E	9.454E	138
139	6.200E-05	2.16E	8.243E	139

 * MOD-5 *

EXAMPLE NUMBER 2
 STUDY OF SLOWING DOWN OF NEUTRONS FROM A 0.5 MICROSECOND, 2.46
 MEV PULSED SOURCE (D,D). DEMONSTRATES TRAVELING ARRAY.

NUMBER OF REAL TRANSIENT STATES 71
 NUMBER OF VIRTUAL TRANSIENT STATES 109
 NUMBER OF SCATTERING ISOTOPIES 1
 NUMBER OF FISSION ISOTOPIES 0
 ROW GENERATION CRITERION 0.0010
 ARRAY GENERATION CRITERION 0.5000
 ARRAY SQUARING CRITERION 0.2500

LOGICALS = 0 1 0 0 0 1 1 0 1 0 0 0 0 0 1 0 0 0 0 0 0 0 0 0 0

STATE WIDTH OPTIMIZED FOR THE ISOTOPE CARBON

-----SYSTEM PARAMETERS-----

ISOTOPE	ATOMIC MASS	MASS DENSITY	NUCLEAR DENSITY	ALPHA
CARBON	12.011	1.6400	0.08224	0.7162

SYSTEM BUCKLING = 0.0 /SQ.CM.

-----NORMALIZED TRANSFER COEFFICIENTS FOR INELASTIC SCATTERING-----

		H(I,J)											
		1	2	3	4	5	6	7	8	9	10	11	12
CARBON -----													
NEUTRON YIELD FROM INELASTIC SCATTERING IN GROUP 1 = 1.00													
1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
2	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
3	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
4	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
5	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
6	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
7	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
9	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
10	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

GENERATING ROUTINE IS BEING CALLED FOR 8 ITERATIONS BEGINNING AT TIME DDT = 6.672E-12 SECONDS.
DDT IS DEFINED FOR VIRTUAL STATE 2 FOR WHICH THE DECAY CONSTANT IS 2.927E 08.

---AT OPERATING ITERATION NO. 1 AND TIME = 0.0 + 0.0 SECONDS, STATE 1.
THE NEUTRON DENSITY (LETHARGY) VECTOR IS WRITTEN BEGINNING WITH VIRTUAL STATE 1.

1	0.0	2.485E-02	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
21	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
31	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
41	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
51	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
61	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
71	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

ABSORBING STATE POPULATIONS 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0

MEAN ENERGY (DENSITY AVERAGED) = 2.2857E 06 MEAN VELOCITY = 2.0916E 09
R.M.S. ENERGY = 2.2857E 06 R.M.S. VELOCITY = 2.0916E 09
RELATIVE STANDARD DEVIATION = 0.0 RELATIVE STANDARD DEVIATION = 0.0

NT= 1 NTIME= 1 IPEAK= 2 ZA= 6.500E 00 ZX= 0.0

---AT OPERATING ITERATION NO. 2 AND TIME = 0.0 + 0.171E-08 SECONDS, STATE 1.
THE NEUTRON DENSITY (LETHARGY) VECTOR IS WRITTEN BEGINNING WITH VIRTUAL STATE 1.

1	0.0	4.167E-02	3.550E-03	3.490E-03	6.180E-04	3.081E-04	4.481E-05	1.540E-05	2.658E-06	6.978E-07	4.023E-15
11	0.0	2.221E-08	3.365E-09	6.140E-10	9.021E-11	1.240E-11	1.516E-12	2.549E-13	2.804E-14	4.023E-15	1.621E-25
21	1.009E-16	7.109E-28	4.743E-18	5.241E-19	4.451E-20	4.349E-21	3.351E-22	2.972E-23	2.055E-24	1.621E-25	3.262E-38
31	1.161E-39	4.817E-41	1.542E-42	2.571E-43	1.323E-44	7.531E-45	3.498E-46	1.790E-47	7.086E-48	3.262E-38	1.186E-71
41	3.777E-55	9.303E-57	1.356E-58	3.007E-60	1.665E-62	5.640E-64	1.465E-66	4.469E-67	8.348E-70	1.186E-71	0.0
51	4.417E-74	6.185E-76	0.0	0.0	0.0	7.713E-64	8.708E-66	1.560E-67	0.0	0.0	0.0
61	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

----- AT OPERATING ITERATION NUMBER 154 THE ARRAY HAS BEEN SQUARED

***AT OPERATING ITERATION 186 THE ARRAY SIZE HAS BEEN REDUCED FROM 71 TO

-----STATE PARAMETERS-----

STATE	II	DU	F	EII	DE	V	VII	DV	RR	NG	STATE
1	0	5	0	0	0	0	0	0	0	4	1
2	0	5	0	0	0	0	0	0	0	4	2
3	0	5	0	0	0	0	0	0	0	4	3
4	0	5	0	0	0	0	0	0	0	4	4
5	0	5	0	0	0	0	0	0	0	4	5
6	0	5	0	0	0	0	0	0	0	4	6
7	0	5	0	0	0	0	0	0	0	4	7
8	0	5	0	0	0	0	0	0	0	4	8
9	0	5	0	0	0	0	0	0	0	4	9
10	0	5	0	0	0	0	0	0	0	4	0
11	0	5	0	0	0	0	0	0	0	4	1
12	0	5	0	0	0	0	0	0	0	4	2
13	0	5	0	0	0	0	0	0	0	4	3
14	0	5	0	0	0	0	0	0	0	4	4
15	0	5	0	0	0	0	0	0	0	4	5
16	0	5	0	0	0	0	0	0	0	4	6
17	0	5	0	0	0	0	0	0	0	4	7
18	0	5	0	0	0	0	0	0	0	4	8
19	0	5	0	0	0	0	0	0	0	4	9
20	0	5	0	0	0	0	0	0	0	4	0
21	0	5	0	0	0	0	0	0	0	4	1
22	0	5	0	0	0	0	0	0	0	4	2
23	0	5	0	0	0	0	0	0	0	4	3
24	0	5	0	0	0	0	0	0	0	4	4
25	0	5	0	0	0	0	0	0	0	4	5
26	0	5	0	0	0	0	0	0	0	4	6
27	0	5	0	0	0	0	0	0	0	4	7
28	0	5	0	0	0	0	0	0	0	4	8
29	0	5	0	0	0	0	0	0	0	4	9
30	0	5	0	0	0	0	0	0	0	4	0
31	0	5	0	0	0	0	0	0	0	4	1
32	0	5	0	0	0	0	0	0	0	4	2
33	0	5	0	0	0	0	0	0	0	4	3
34	0	5	0	0	0	0	0	0	0	4	4
35	0	5	0	0	0	0	0	0	0	4	5
36	0	5	0	0	0	0	0	0	0	4	6
37	0	5	0	0	0	0	0	0	0	4	7
38	0	5	0	0	0	0	0	0	0	4	8
39	0	5	0	0	0	0	0	0	0	4	9
40	0	5	0	0	0	0	0	0	0	4	0
41	0	5	0	0	0	0	0	0	0	4	1
42	0	5	0	0	0	0	0	0	0	4	2
43	0	5	0	0	0	0	0	0	0	4	3
44	0	5	0	0	0	0	0	0	0	4	4
45	0	5	0	0	0	0	0	0	0	4	5
46	0	5	0	0	0	0	0	0	0	4	6
47	0	5	0	0	0	0	0	0	0	4	7
48	0	5	0	0	0	0	0	0	0	4	8
49	0	5	0	0	0	0	0	0	0	4	9
50	0	5	0	0	0	0	0	0	0	4	0
51	0	5	0	0	0	0	0	0	0	4	1
52	0	5	0	0							

----- A MOD5 CALCULATION -----

EXAMPLE NUMBER 2
STUDY OF SLOWING DOWN OF NEUTRONS FROM A 0.5 MICROSECOND, 2.46 MEV PULSED SOURCE (D,D). DEMONSTRATES TRAVELING ARRAY.

THE CENTRAL CORE FLUX-AVERAGED ENERGY IS 2.911E 05 EV.
THE LEAKAGE FLUX-AVERAGED ENERGY IS 6.00E 05 EV.

AT	9.655E-	5 SECONDS -	NEUTRONS	HAVE	SUFFERED NON-FISSION CAPTURE,
0.000	PERCENT OF THE	NEUTRONS	HAVE	LEAKED FROM THE SYSTEM,	
0.000	PERCENT OF THE	NEUTRONS	HAVE	CAUSED FISSION,	
97.965	PERCENT ARE STILL SLOWING DOWN,	AND	ENERGY BELOW 1.0E 00 EV,		
0.000	PERCENT HAVE BEEN PASSED BY THE ARRAY.				
2.035					
0.113					

***** FINAL STATE VECTOR

[illegible]

***** FINAL CORE FLUX VECTOR

[illegible]

FINAL SURFACE FLUX VECTOR

The image displays a sequence of 10 diagrams, each representing a stage in the construction of a 10x10 grid of circles. The diagrams are arranged vertically. Each diagram consists of two rows of circles, with dots placed between them to indicate the positions of the circles. The sequence shows the grid growing from a single circle in the first diagram to a full 10x10 grid in the tenth diagram. The circles are arranged in a regular pattern, and the dots are placed at the intersections of the grid lines.

EXAMPLE NUMBER 3
STUDY OF SLOWING DOWN SPECTRA IN REPRESENTATIVE FAST MULTIPLY-
ING ASSEMBLY. FISSION SOURCE - DELTA FUNCTION IN TIME.

NUMBER OF REAL-VALUE TRANSIENT STATES	71
NUMBER OF CHARACTER TRANSIENT STATES	121
NUMBER OF SCALAR TRANSIENT STATES	4
NUMBER OF FISSILE TRANSIENT STATES	2
NUMBER OF FISSILE TRANSIENT STATES	0.0010
NUMBER OF FISSILE TRANSIENT STATES	0.4000
NUMBER OF FISSILE TRANSIENT STATES	0.2000

LOGICALS = 01001000010000010000000000000000

-----SYSTEM PARAMETERS-----

ISOTOPE	ATOMIC MASS	MASS DENSITY	NUCLEAR DENSITY	ALPHA
ALUMINUM	26.980	0.8480	0.01893	0.8621
IRON	55.850	0.9650	0.01041	0.9309
U-235	235.000	2.6220	0.00631	0.9831
U-238	238.000	3.0160	0.00763	0.9833

SYSTEM BUCKLING = 0.007200/SQ.CM.

-----TABLE OF SCALE FACTORS-----

GROUP	1	2	3	4	5
1	1.00	0.50	0.10	0.10	1.00
2	1.00	0.50	0.10	0.10	1.00
3	1.00	0.50	0.10	0.10	1.00
4	1.00	0.50	0.10	0.10	1.00
5	1.00	0.50	0.10	0.10	1.00
6	1.00	0.50	0.10	0.10	1.00
7	1.00	0.50	0.10	0.10	1.00
8	1.00	0.50	0.10	0.10	1.00
9	1.00	0.50	0.10	0.10	1.00
10	1.00	0.50	0.10	0.10	1.00
11	1.00	0.50	0.10	0.10	1.00
12	1.00	0.50	0.10	0.10	1.00
13	1.00	0.50	0.10	0.10	1.00
14	1.00	0.50	0.10	0.10	1.00
15	1.00	0.50	0.10	0.10	1.00
16	1.00	0.50	0.10	0.10	1.00
17	1.00	0.50	0.10	0.10	1.00
18	1.00	0.50	0.10	0.10	1.00
19	1.00	0.50	0.10	0.10	1.00
20	1.00	0.50	0.10	0.10	1.00
21	1.00	0.50	0.10	0.10	1.00
22	1.00	0.50	0.10	0.10	1.00
23	1.00	0.50	0.10	0.10	1.00
24	1.00	0.50	0.10	0.10	1.00
25	1.00	0.50	0.10	0.10	1.00
26	1.00	0.50	0.10	0.10	1.00

-----MACROSCOPIC CROSS SECTIONS-----

GROUP	TOTAL	TRANSPORT	LEAKAGE	ALUMINUM ELASTIC	ALUMINUM CAPTURE	ALUMINUM INELASTIC	ALUMINUM FISSION	ALUMINUM COSINE
1	0.107061	0.090272	0.025314	0.018931	0.001798	0.015145	0.0	0.640000
2	0.112632	0.094960	0.022106	0.027071	0.000435	0.014198	0.0	0.570000
3	0.119265	0.102357	0.021822	0.038808	0.000038	0.012305	0.0	0.470000
4	0.121009	0.133311	0.017357	0.047257	0.000008	0.009465	0.0	0.360000
5	0.122209	0.135009	0.017208	0.057708	0.000008	0.000000	0.0	0.290000
6	0.126482	0.168122	0.013909	0.073779	0.000013	0.000000	0.0	0.200000
7	0.126381	0.192480	0.012189	0.070530	0.000019	0.000000	0.0	0.110000
8	0.148860	0.253762	0.006297	0.088818	0.000051	0.000000	0.0	0.060000
9	0.146171	0.276300	0.008550	0.075695	0.000027	0.000000	0.0	0.040000
10	0.196624	0.277739	0.008316	0.096353	0.000071	0.000000	0.0	0.030000
11	0.102959	0.269083	0.008716	0.091931	0.000019	0.000000	0.0	0.024700
12	0.151897	0.356611	0.006594	0.034197	0.000379	0.000000	0.0	0.024700
13	0.145812	0.351808	0.006738	0.026503	0.000013	0.000000	0.0	0.024700
14	0.173145	0.356509	0.006650	0.026503	0.000019	0.000000	0.0	0.024700
15	0.227198	0.456014	0.005213	0.026503	0.000028	0.000000	0.0	0.024700
16	0.266818	0.458591	0.005184	0.026503	0.000040	0.000000	0.0	0.024700
17	0.338818	0.391726	0.006059	0.026503	0.000059	0.000000	0.0	0.024700
18	0.405463	0.435709	0.005332	0.026503	0.000087	0.000000	0.0	0.024700
19	0.521168	0.462397	0.005142	0.026503	0.000127	0.000000	0.0	0.024700
20	0.630008	0.449679	0.005286	0.026503	0.000189	0.000000	0.0	0.024700

GROUP	IRON ELASTIC	IRON CAPTURE	IRON INELASTIC	IRON FISSION	IRON COSINE	U-235 ELASTIC	U-235 CAPTURE	U-235 INELASTIC	U-235 FISSION	U-235 COSINE
1	0.02071	0.00037	0.01426	0.0	0.83000	0.02352	0.00013	0.00692	0.01176	0.84000
2	0.02550	0.00005	0.01405	0.0	0.72000	0.02890	0.00020	0.01290	0.00773	0.80000
3	0.02446	0.00002	0.01176	0.0	0.45000	0.03024	0.00027	0.01284	0.00840	0.71000
4	0.02436	0.00003	0.00937	0.0	0.32000	0.03021	0.00040	0.01183	0.00860	0.55000
5	0.02525	0.00004	0.00395	0.0	0.24000	0.02587	0.00081	0.00927	0.00840	0.35000
6	0.03637	0.00004	0.00010	0.0	0.17000	0.03226	0.00114	0.00866	0.00827	0.25000
7	0.03024	0.00005	0.0	0.0	0.09000	0.04395	0.00168	0.00672	0.00948	0.20000
8	0.03651	0.00005	0.0	0.0	0.05000	0.05712	0.00269	0.00603	0.01142	0.13000
9	0.05038	0.00006	0.0	0.0	0.03000	0.06465	0.00433	0.00612	0.01411	0.07000
10	0.09909	0.00010	0.0	0.0	0.02000	0.06922	0.00672	0.00040	0.01781	0.04000
11	0.04152	0.00005	0.0	0.0	0.01190	0.07459	0.00992	0.0	0.02557	0.02000
12	0.08501	0.00004	0.0	0.0	0.01190	0.08400	0.01378	0.0	0.03887	0.01000
13	0.06130	0.00011	0.0	0.0	0.01190	0.09895	0.01773	0.0	0.03482	0.00270
14	0.07691	0.00069	0.0	0.0	0.01190	0.10575	0.02359	0.0	0.04532	0.00270
15	0.10427	0.00016	0.0	0.0	0.01190	0.09700	0.03759	0.0	0.06592	0.00270
16	0.11447	0.00029	0.0	0.0	0.01190	0.08400	0.05264	0.0	0.09774	0.00270
17	0.11864	0.00039	0.0	0.0	0.01190	0.08199	0.06908	0.0	0.11596	0.00270
18	0.11864	0.00055	0.0	0.0	0.01190	0.08064	0.09325	0.0	0.16676	0.00270
19	0.11864	0.00075	0.0	0.0	0.01190	0.08064	0.12780	0.0	0.20258	0.00270
20	0.11864	0.00109	0.0	0.0	0.01190	0.08064	0.20006	0.0	0.19703	0.00270

GROUP	U-238 ELASTIC	U-238 CAPTURE	U-238 INELASTIC	U-238 FISSION	U-238 COSINE	ELASTIC	CAPTURE	INELASTIC	FISSION	COSINE
1	0.02671	0.00008	0.01374	0.00763	0.84000	0.0	0.0	0.0	0.0	0.0
2	0.03358	0.00015	0.01916	0.00443	0.80000	0.0	0.0	0.0	0.0	0.0
3	0.03282	0.00046	0.01984	0.00443	0.71000	0.0	0.0	0.0	0.0	0.0
4	0.03511	0.00099	0.01717	0.00374	0.53000	0.0	0.0	0.0	0.0	0.0
5	0.04595	0.00099	0.01641	0.00015	0.42000	0.0	0.0	0.0	0.0	0.0
6	0.06411	0.00114	0.01259	0.0	0.33000	0.0	0.0	0.0	0.0	0.0
7	0.08167	0.00114	0.00801	0.0	0.21000	0.0	0.0	0.0	0.0	0.0
8	0.09388	0.00267	0.00420	0.0	0.12000	0.0	0.0	0.0	0.0	0.0
9	0.10228	0.00350	0.00145	0.0	0.07000	0.0	0.0	0.0	0.0	0.0
10	0.11053	0.00452	0.0	0.0	0.04000	0.0	0.0	0.0	0.0	0.0
11	0.08896	0.00569	0.0	0.0	0.02000	0.0	0.0	0.0	0.0	0.0
12	0.08896	0.00815	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
13	0.11209	0.01244	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
14	0.11664	0.01839	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
15	0.08605	0.01921	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
16	0.15891	0.03736	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
17	0.09951	0.02964	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
18	0.16920	0.07396	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
19	0.12030	0.13044	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0
20	0.12030	0.13044	0.0	0.0	0.00270	0.0	0.0	0.0	0.0	0.0

***** (N,2N) REACTIONS OCCUR IN IRON . ROW SUM ERRORS WILL NOT STOP EXECUTION OF THIS PROBLEM
 ***** (N,2N) REACTIONS OCCUR IN U-235 . ROW SUM ERRORS WILL NOT STOP EXECUTION OF THIS PROBLEM
 ***** (N,2N) REACTIONS OCCUR IN U-238 . ROW SUM ERRORS WILL NOT STOP EXECUTION OF THIS PROBLEM

-----NORMALIZED TRANSFER COEFFICIENTS FOR INELASTIC SCATTERING-----

H(I,J)

I	1	2	3	4	5	6	7	8	9	10	11	12
-----ALUMINUM-----												
NEUTRON YIELD FROM INELASTIC SCATTERING IN GROUP 1 = 1.00												
1	0.012	0.075	0.187	0.313	0.225	0.125	0.050	0.012	0.0	0.0	0.0	0.0
2	0.0	0.0	0.347	0.293	0.147	0.093	0.040	0.013	0.0	0.0	0.0	0.0
3	0.0	0.0	0.0	0.477	0.200	0.154	0.062	0.015	0.0	0.0	0.0	0.0
4	0.0	0.0	0.0	0.0	0.540	0.380	0.0	0.0	0.0	0.0	0.0	0.0
5	0.0	0.0	0.0	0.0	0.0	0.231	0.462	0.231	0.077	0.0	0.0	0.0
6	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
7	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
9	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
10	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

-----IRON-----												
NEUTRON YIELD FROM INELASTIC SCATTERING IN GROUP 1 = 1.00												
1	0.007	0.058	0.139	0.292	0.248	0.161	0.058	0.029	0.007	0.0	0.0	0.0
2	0.0	0.0	0.163	0.230	0.244	0.170	0.074	0.022	0.0	0.0	0.0	0.0
3	0.0	0.0	0.0	0.531	0.124	0.115	0.044	0.011	0.0	0.0	0.0	0.0
4	0.0	0.0	0.0	0.0	0.511	0.278	0.033	0.0135	0.0	0.0	0.0	0.0
5	0.0	0.0	0.0	0.0	0.0	0.405	0.0	0.0	0.027	0.0	0.0	0.0
6	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
7	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
9	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
10	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

-----U-235-----												
NEUTRON YIELD FROM INELASTIC SCATTERING IN GROUP 1 = 1.79												
1	0.0	0.010	0.049	0.243	0.417	0.544	0.340	0.136	0.039	0.010	0.0	0.0
2	0.0	0.0	0.042	0.182	0.281	0.266	0.135	0.057	0.021	0.005	0.0	0.0
3	0.0	0.0	0.0	0.141	0.277	0.283	0.157	0.063	0.024	0.005	0.0	0.0
4	0.0	0.0	0.0	0.0	0.199	0.320	0.227	0.091	0.038	0.011	0.0	0.0
5	0.0	0.0	0.0	0.0	0.145	0.370	0.268	0.145	0.067	0.017	0.0	0.0
6	0.0	0.0	0.0	0.0	0.0	0.367	0.367	0.183	0.010	0.033	0.0	0.0
7	0.0	0.0	0.0	0.0	0.0	0.0	0.610	0.350	0.483	0.117	0.0	0.0
8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.500	0.133	0.0	0.0
9	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
10	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

-----U-238-----												
NEUTRON YIELD FROM INELASTIC SCATTERING IN GROUP 1 = 1.44												
1	0.0	0.006	0.041	0.228	0.361	0.417	0.239	0.089	0.038	0.011	0.0	0.0
2	0.0	0.0	0.046	0.159	0.279	0.291	0.151	0.052	0.016	0.004	0.0	0.0
3	0.0	0.0	0.023	0.112	0.258	0.264	0.120	0.077	0.023	0.004	0.0	0.0
4	0.0	0.0	0.0	0.062	0.258	0.264	0.158	0.084	0.027	0.005	0.0	0.0
5	0.0	0.0	0.0	0.0	0.335	0.294	0.188	0.160	0.014	0.006	0.0	0.0
6	0.0	0.0	0.0	0.0	0.0	0.0	0.105	0.176	0.019	0.006	0.0	0.0
7	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.182	0.041	0.006	0.0	0.0
8	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.158	0.006	0.0	0.0
9	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
10	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
11	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
12	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0

ROW SUMS, BEGINNING WITH ROW 1

[illegible]

-----SOURCE VECTOR POP(I,1) (NEUTRONS PER STATE)-----

[illegible]

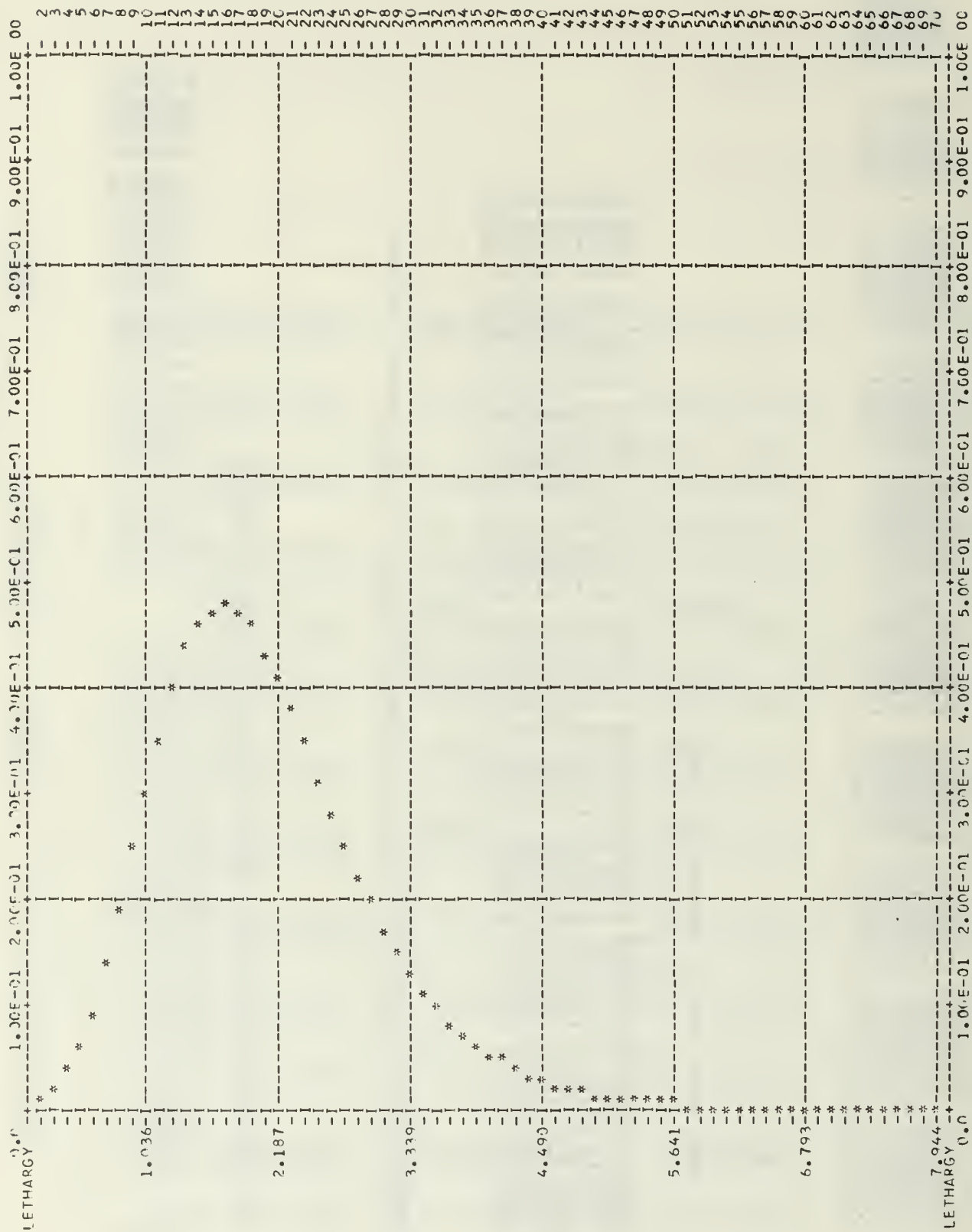
GENERATING ROUTINE IS BEING CALLED FOR 8 ITERATIONS BEGINNING AT TIME DDT = 6.644E-12 SECONDS.
DDT IS DEFINED FOR VIRTUAL STATE 16 FOR WHICH THE DECAY CONSTANT IS 2.352E 08.

-----AT UPDATING ITERATION NO. 1 AND TIME = 0.0 + 0.0 SECONDS, STATE 1.
THE NEUTRON DENSITY (LETHARGY) VECTOR IS WRITTEN BEGINNING WITH VIRTUAL STATE 1.

[illegible]

ABSORBING STATE POPULATIONS

MEAN ENERGY (CENTS)	1.9675E 06	MEAN VELOCITY	1.7951E 09
STDEV. ENERGY	2.4947E 06	R.M.S. VELOCITY	1.9405E 09
RELATIVE STANDARD DEVIATION	7.7955E 01	RELATIVE STANDARD DEVIATION	4.0886E 01



---AT OPERATING ITERATION NO. 24 AND TIME = 3.0 * 0.493E-07 SECONDS, VECTOR IS WRITTEN BEGINNING WITH VIRTUAL STATE 1.

1	0.294E-06	2.316E-12	2.248E-11	1.452E-10	7.690E-10	4.846E-09	1.546E-08	6.094E-08	2.002E-07	8.686E-07
11	2.712E-06	1.604E-05	5.144E-05	1.271E-04	2.939E-04	5.510E-04	8.964E-04	2.411E-03	6.216E-03	
21	1.266E-02	2.619E-02	4.193E-02	5.105E-02	7.453E-02	9.663E-02	1.135E-01	1.243E-01	1.289E-01	
31	1.384E-01	1.397E-01	1.382E-01	1.371E-01	1.262E-01	9.535E-02	9.179E-02	8.912E-02	8.741E-02	
41	8.419E-02	7.361E-02	4.238E-02	3.525E-02	3.205E-02	3.050E-02	2.977E-02	1.971E-02	1.148E-02	
51	9.765E-03	9.789E-03	8.328E-03	7.316E-03	5.620E-03	2.097E-03	1.477E-03	1.273E-03	1.149E-03	
61	1.647E-03	7.177E-04	3.907E-04	3.197E-04	2.738E-04	2.334E-04	1.987E-04	1.621E-04	1.422E-04	
71	4.526E-02									

ABSORBING STATE POPULATIONS 4.026E-06 5.516E-02 4.070E-01 2.678E-01

MEAN ENERGY (DENSITY AVERAGED) = 3.0153E 05
 R.M.S. ENERGY = 3.7373E 05
 RELATIVE STANDARD DEVIATION = 7.3212E-01
 MEAN VELOCITY = 7.1180E 08
 R.M.S. VELOCITY = 7.5964E 08
 RELATIVE STANDARD DEVIATION = 3.7274E-01

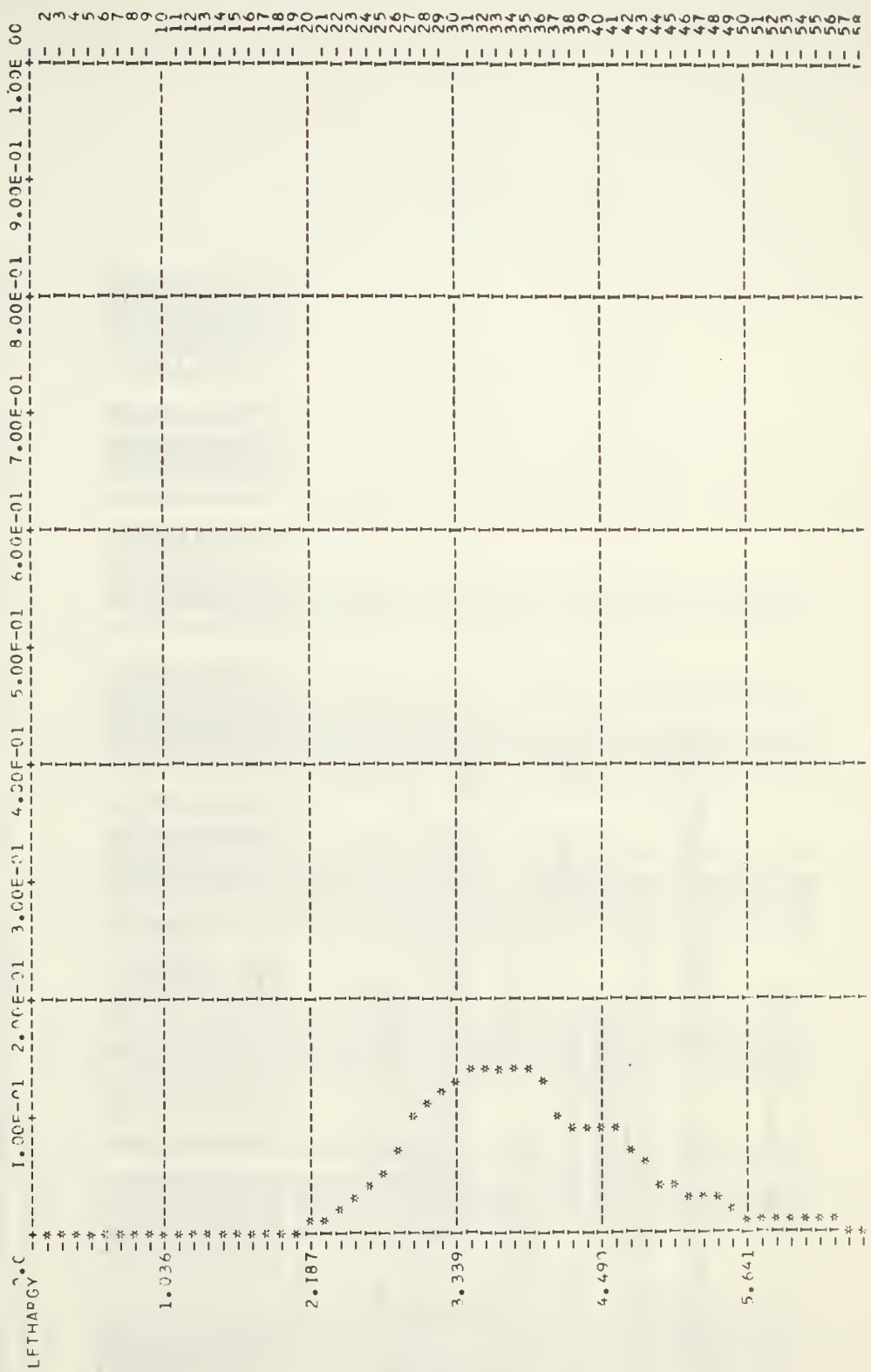


TABLE OF MOMENTS OF DISTRIBUTIONS
(UNITS ARE MICROSECONDS)

N	ABSORBING STATE MOMENTS			TRUNCATION CORRECTIONS		
	FISSION	LEAKAGE	CAPTURE	FISSION	LEAKAGE	CAPTURE
1	4.7045E-03	3.3793E-02	7.5464E-02	4.9540E-03	1.5838E-03	1.1142E-02
2	7.4554E-03	3.5977E-03	1.3548E-02	2.2493E-03	6.6866E-04	5.4448E-03
RSD	1.4751E-04	1.4664E-00	1.2038E-00	1.4700E-00		

INTERPOLATED DISTRIBUTIONS
 $\Delta t = 2.0E-09$ SECONDS

N	TIME	FISSION	LEAKAGE	CAPTURE	TOTAL	N
1	0.00E-10	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	1
2	0.00E-09	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	2
3	0.00E-08	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	3
4	0.00E-07	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	4
5	0.00E-06	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	5
6	0.00E-05	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	6
7	0.00E-04	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	7
8	0.00E-03	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	8
9	0.00E-02	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	9
10	0.00E-01	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	10
11	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	11
12	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	12
13	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	13
14	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	14
15	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	15
16	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	16
17	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	17
18	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	18
19	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	19
20	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	20
21	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	21
22	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	22
23	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	23
24	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	24
25	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	25
26	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	26
27	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	27
28	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	28
29	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	29
30	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	30
31	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	31
32	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	32
33	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	33
34	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	34
35	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	35
36	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	36
37	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	37
38	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	38
39	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	39
40	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	40
41	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	41
42	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	42
43	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	43
44	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	44
45	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	45
46	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	46
47	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	47
48	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	48
49	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	49
50	0.00E-00	5.5336E-07	5.869E-07	2.073E-07	5.188E-07	50

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13. ABSTRACT

This document provides users' information for a computer code, MOD-5, which calculates the time and energy dependent evolution of the neutron density in homogeneous media following initiation of a pulsed neutron source of arbitrary energy distribution. The code is based on a discrete stochastic model of the neutron slowing down process developed by the author. Copies of the code and associated computer software are available through the Argonne Code Center, Argonne National Laboratory, 9700 South Cass Avenue, Argonne, Illinois 60439.

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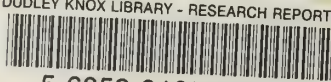
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